

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) **DIABLO CANYON UNIT 1** DOCKET NUMBER (2) **05000275** PAGE (3) **1 OF 2**

TITLE (4) **REACTOR COOLANT SYSTEM LOOP LOW FLOW REACTOR TRIP**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0	1	02	85	001	00	02	01	85		05000

OPERATING MODE (9) **1** THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

POWER LEVEL (10) 49	<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.408(e)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
	<input type="checkbox"/> 20.408(a)(1)(i)	<input type="checkbox"/> 80.38(e)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)
	<input type="checkbox"/> 20.408(a)(1)(ii)	<input type="checkbox"/> 80.38(e)(2)	<input type="checkbox"/> 80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
	<input type="checkbox"/> 20.408(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)	
	<input type="checkbox"/> 20.408(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(vii)(B)	
	<input type="checkbox"/> 20.408(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **DAVID P. SISK, REGULATORY COMPLIANCE ENGINEER** TELEPHONE NUMBER **805 595-7351**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	TIG	FISV	PJO70	N					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH **01** DAY **04** YEAR **85**

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While in Mode 1 (Power Operation), the reactor tripped on reactor coolant system (RCS) low flow when a solenoid valve failure in the turbine overspeed protection controller (TOPC) system caused the turbine to slow with a corresponding drop in RCS flow. Diesel Generator 1-2 autostarted but did not load. While shifting to startup power, two of five containment fan cooler units, CFCU 1-1 and 1-4 tripped on thermal overload after autostarting on high speed.

The plant was stabilized in Mode 3 (hot standby) in accordance with procedures. All systems and equipment affected by this event were returned to normal operation. SV-41 was replaced with a spare and tested satisfactorily on January 4, 1985.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) DIABLO CANYON UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 5	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 0 1	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1945 PST, January 2, 1985, with Unit 1 in Mode 1 (Power Operation), low flow in the Reactor Coolant System (AB) initiated a reactor trip during the performance of Start Up Procedure 43.7, "Net Load Trip From 50 Percent Power." A solenoid valve in the Turbine Overspeed Protection Controller (TOPC) System caused the turbine (TA) (TRB) to slow with a corresponding drop in RCS flow. All automatic equipment responded as designed. Diesel Generator 1-2 autostarted but did not load. When shifting to startup power, two of five Containment Fan Cooler Units (BK) (FCU), CFCU 1-1 and 1-4 tripped on thermal overload after autostarting on high speed. They were manually started in low speed. The high speed windings were reset and both CFCUs were successfully started on high speed. Operations personnel blocked the Low Pressure Safety Injection (LPSI) signal to prevent an unnecessary safety injection actuation from post trip conditions, i.e., steam demand from the Auxiliary Steam System and Main Turbine Pilot Trip Valve leakage. The LPSI setpoint was never reached and the block was reset when plant conditions were stabilized.

The trip was caused by a solenoid valve failure in the Turbine Overspeed Protection Controller (TOPC) System. This system causes the turbine governor (TA) (FCV) and intercept (TA) (ISV) valves to close for short periods to prevent turbine overspeed during transients. TOPC Solenoid Valve SV-41 stuck open during the load rejection transient. This prevented the governor (TA) (FCV) and intercept (TA) (ISV) valves from reopening, thus allowing the Turbine (TA) (TRB) to slow, and generator frequency to approach 54 Hertz. The reactor coolant pump's speed dropped as frequency dropped. This lowered flow in the reactor coolant system causing an RCS loop low flow reactor trip.

The plant was stabilized in Mode 3 (Hot Standby) in accordance with procedures. All systems and equipment affected by this event were returned to normal operation. SV-41 was replaced with a spare and tested satisfactorily on January 4, 1985. Start Up Test 43.7 was completed satisfactorily on January 5, 1985.

This event was a previously analyzed Condition II event and had no effect on the health and safety of the public.

PACIFIC GAS AND ELECTRIC COMPANY

PG&E

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

February 1, 1985

PGandE Letter No.: DCL-85-044

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Diablo Canyon Unit 1
Licensee Event Report 85-001-00
ESF Actuation - Reactor Trip

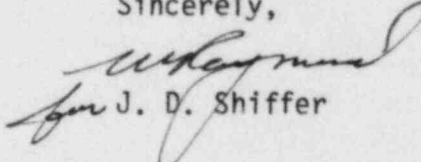
Gentlemen:

Pursuant to 10 CFR 50.73(a)(2)(iv), PGandE is submitting the enclosed Licensee Event Report concerning the inadvertent actuation of an engineered safety feature, reactor trip.

This event has in no way affected the public's health and safety.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelop.

Sincerely,


for J. D. Shiffer

Enclosure

cc: J. B. Martin
Service List

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