

LICENSEE EVENT REPORT (LER)

APPROVED DATE NO. 318-8108
EXPIRES - 8/31/85

FACILITY NAME (1)

INDIAN POINT UNIT 2

DOCKET NUMBER (2)

0 5 0 0 0 1 2 4 7

PAGE (3)

1 OF 12

TITLE (4)

SPURIOUS SAFETY INJECTION SIGNAL

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (8)
12	28	84	84	026	0	12	28	85		0 5 0 0 0 1 1
										0 5 0 0 0 1 1

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § 170.46 AND ONE OR MORE OF THE FOLLOWING (11)									
POWER LEVEL (10)	28.402(a)	28.406(a)	80.73(a)(2)(iii)	72.71(a)						
	28.406(a)(1)(i)(B)	80.73(a)(1)	80.73(a)(2)(iv)	72.71(b)						
	28.406(a)(1)(i)(B)	80.73(a)(2)	80.73(a)(2)(v)	OTHER (Specify in Appendix B and in Part, NRC Form 266A)						
	28.406(a)(1)(i)(B)	80.73(a)(2)(i)	80.73(a)(2)(vi)(A)							
	28.406(a)(1)(i)(B)	80.73(a)(2)(ii)	80.73(a)(2)(vi)(B)							
	28.406(a)(1)(i)(v)	X 80.73(a)(2)(iii)	80.73(a)(2)(v)							

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
JOHN ELLWANGER	91114 92161-191182

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If you complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/>	<input checked="" type="checkbox"/>				

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-spaced typewritten lines) (16)

On December 28, 1984, while the reactor was subcritical and an orderly cooldown in progress, a spurious Safety Injection signal occurred. All three Safety Injection pumps were de-energized due to a previous incident. The reactor coolant system was at 345°F and 2000 psia.

At 7:30 p.m. one instrument channel monitoring steam flow from Steam Generator #24 was observed to be drifting excessively in the high (conservative) direction. At 10:20 p.m. a high steam flow signal was generated from #23 Steam Generator. For the existing reactor coolant temperature and pressure, a second high steam flow signal was sufficient to result in a Safety Injection signal.

The behavior of the two channels is not unique although the magnitude of the drift was excessive. A drift of this magnitude is considered to be an isolated case. There was no impact on plant safety as the Safety Injection pumps would have normally been de-energized for this reactor coolant temperature and pressure.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)		PAGE (3)	
		YEAR	REVISION NUMBER		
		SEQUENTIAL NUMBER	REVISION NUMBER		
INDIAN POINT UNIT 2	0 15 10 10 10 12 14 7	814	- 01216	- 010 012	OF 012

TEXT (1) more space is required, use additional NRC Form 266A (17)

On December 28, 1984, while the reactor was subcritical and an orderly cooldown in progress, a spurious Safety Injection signal occurred. All three Safety Injection pumps were de-energized due to a previous incident (LER 84-025). The reactor coolant system was at 345°F and 2000 psia.

At 7:30 p.m. one instrument channel monitoring steam flow from Steam Generator #24 was observed to be drifting excessively in the high (conservative) direction. The operator placed this channel in the tripped condition in accordance with procedures. At 10:20 p.m. a high momentary steam flow signal was generated from #23 Steam Generator line. For the existing reactor coolant system temperature and pressure, the two high steam flow signals were sufficient to result in a Safety Injection signal.

The instrumentation loop associated with the first channel drift was checked and found to be satisfactory. The transmitter sensing tubes were blown down, vented and placed backed in service.

The response of the steam flow channels under these conditions is due to inherent non-linearity of errors which become significant at lower scale ranges. The setpoint, associated with high steam flow, is constant from zero to 20% load. For the steam generator conditions at the time of the second high steam flow signal, a mass flow rate of 14.8% of full steam flow would have been sufficient to produce the same ΔP as for full steam flow. Steam flow signals are susceptible to system errors at the low (less than 25% full scale) end of the scale. These errors are due to instrument drift, air in impulse lines and electronic instability. No further action is planned.

There was no impact upon plant safety as both High Flow channels were activated by steam flows which are below the minimum required. From the viewpoint of plant safety, the behavior of both instrument channels was conservative.

John D. O'Toole
Vice President

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January 28, 1985

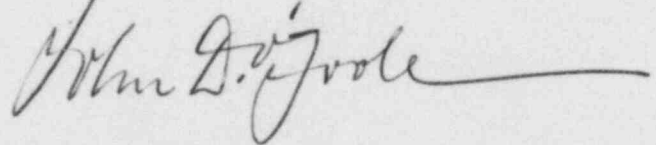
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-84-026-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-84-026-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
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