

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104 EXPIRES - 8/31/93

FACILITY NAME (1) **Limerick Generating Station - Unit 1** DOCKET NUMBER (2) **050103521** PAGE (3) **1** OF **2**

TITLE (4) **Reactor Water Cleanup Isolation**

EVENT DATE (6)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
01	01	1985	85	001	0	01	01	1985		050103521
										050103521

OPERATING MODE (9) **2**

POWER LEVEL (10) **000**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input checked="" type="checkbox"/> 60.736(2)(i)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 60.736(1)	<input type="checkbox"/> 60.736(2)(i)	<input type="checkbox"/> 73.71(d)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 60.736(2)	<input type="checkbox"/> 60.736(2)(ii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 60.736(2)(i)	<input type="checkbox"/> 60.736(2)(iii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 60.736(2)(ii)	<input type="checkbox"/> 60.736(2)(iii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 60.736(2)(iii)	<input type="checkbox"/> 60.736(2)(iv)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **John C. Nagle, Engineer - Special Projects** TELEPHONE NUMBER **215 841-5184**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	CE	11TISR	121718	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 10 lines single-space typewritten lines) (16)

Abstract: 85-001

On January 1, 1985, the reactor water cleanup system outboard isolation valve closed to the isolation position upon receiving a signal from the high ambient temperature logic circuit of the Reactor Water Cleanup portion of the Nuclear Steam Supply Shutoff System (NSSSS). The high ambient temperature parameter functions to detect a Reactor Water Cleanup system leak by monitoring the temperatures in the Reactor Water Cleanup rooms. The isolation signal was unexpectedly generated by setting an ambient Temperature Transmitter Switch to the "READ" position. The isolation signal was cleared, and the reactor water cleanup system was returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 0 1	- 0 0	0 2	OF	0 2

TEXT (if more space is required, use additional NRC Form 368A (17))

Description of the Event:

On January 1, 1985, at 1:24 a.m., the reactor water cleanup system outboard isolation valve, HV-44-1F004, received a signal from the high ambient temperature logic circuit of the Reactor Water Cleanup portion of NSSSS. HV-44-1F004 closed to the isolation position. After the high ambient temperature isolation signal was cleared, the reactor water cleanup system was returned to normal operation.

Consequences of the Event:

The reactor water cleanup system isolated properly upon receiving the false high ambient temperature signal. There were no adverse consequences. In addition, reactor water chemistry, because of the short duration of the isolation, was not adversely affected.

Cause of the Event:

During performance of a surveillance test, setting high ambient Temperature Transmitter Switch (TTS-44-1N600D) to the "READ" position caused a spurious isolation. A defect in the ambient temperature transmitter switch is believed to be responsible for the spurious isolation signal.

Corrective Actions:

A modification is being pursued to the "READ" circuit which will prevent inadvertent trips when using the "READ" switch on the temperature switch.

Previous Similar Occurrences:

LGS LERs 84-012, 84-026, 84-034, 84-035, 84-036.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 641-4000

January 31, 1985

Docket No. 50-352

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Washington, DC 20555

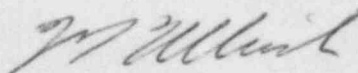
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the automatic isolation of the reactor water cleanup system.

Reference: Docket No. 50-352
Report Number: 85-001
Revision Number: 00
Event Date: January 1, 1985
Report Date: January 31, 1985
Facility: Limerick Generating Station
P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10CFR50.73 (a) (2) (iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

IE22
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1/16/85