NRC Form 388 (9-43) LICENSEE EVENT REPORT (LER)								U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85								
PACILIT	Y NAME (1)										DOCKET NUMBE	R (2)		1 17	AGE (3)
	Dr	esde	a Nuc	lear Powe	er S	tati	on		729			0 5 0 0		21 419	1 0	F 0 12
TITLE (4	•	acto														
ev	ENT DATE			LER NUMBER				PORT DAT	TE (7)		OTHER	FACILITIES INV	OLVED I	8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIA	-	REVISION NUMBER	MONTH	DAY	YEAR		FACILITY NAM	ues		ET NUMBE		
	H					7					N/A		0	51010	101	11
1 0	1 4	8 4	8 4	0116	6 -	0 1	0 1	2 3	8 4		N/A		0.	5 10 10	101	1.1
	RATING	1,,	THIS RE	PORT IS SUBMITT	10 PU	REUANT 1		EQUIREM	INTE OF 1	0 CFR 9: /C	Check one or more	of the following) (_			
	OOE (9)	N		.402(b)			20,4(6)			Х	60.73(a)(2)(lv)		H	73,71(b)		
LEVE (10)	L	19 1 0		.406(a)(1)(i)		-	80,35(a)(1) 80,36(a)(2)			H	90.73(a)(2)(v) 90.73(a)(2)(vii)		H	73.71(e) OTHER /S		
		7 0		.405(a)(1)(M)			50.73(a)			H	90,73(a)(2)(viii)(/	A)	H	below and I		
			20.	.406(a)(1)(iv)			£1,75%				90,73(a)(2)(vili)(1					
			20.	.406(a)(1)(v)	_		94.7 Sin)				80.73(a)(2)(x)	electron 1				
NAME							ICENSEE (CONTACT	FOR THIS	LER (12)			TELEP	HONE NUM	RER	
												AREA CODE	T			leg et
	La	wren	ce Bo	yle		(X-5	26)			I, Holis	4 15 15	81115	914	12 1-	1219	1210
				COMPLETE	ONEL	INE POR	EACH CO	MPONENT	FAILURE	DESCRIBE	D IN THIS REPOR	T (13)	_			*********
CAUSE	SYSTEM	сомро	NENT	MANUFAC- TURER		NPROS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER		NPROS		
х	SIH	0 1 01	210	C161710	O N					1	111	111				
х	SIH	0 0	210	P1 314 1 0	N						111	ш				
				BUPPLEM	ENTAL	REPORT	EXPECTE	0 (14)				EXPECT		MONTH	DAY	YEAR
TYE	(If yes, or	omplete EX	PECTED	SUBMISSION DAT			V	00 D				SUBMISS DATE	ION			
				oproximetely fifteen		spece type	written line									11
	va lo Sa	cramme icuum ow ste	afte am s sign	erming DOF ne to low er the vac seal heade dificance This is t	con cuum er p was	dense bres ressu	er va aker ure b	cuum. was c ecaus	The opened se the	to postean	tor scram revent tu m seal va or scram	mmed agai irbine da ilve (S-1 function	in at mage) wo	due ould n	to ot of	pen.

8502060408 850123 PDR ADDCK 05000249 PDR IE 22

NRC Form 368A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

		EXPIRES: 8/31	85		
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)		
		YEAR SEQUENTIAL REVISION NUMBER NUMBER			
Dresden Nuclear Power Station	0 5 0 0 0 2 4	9 814 - 011 6 - 9 1	Q 2 OF 012		

Low condenser vacuum was caused by the 3-4402-A valve not opening completely while performing the Circulating Water Flow Reversal Test, DOP 4400-8. An attempt was made to reverse flow but the low condenser vacuum reactor scram actuated before this could be accomplished. After the Circulating Water Flow Reversal evolution was completed, condenser vacuum returned to normal and the initial reactor scram was reset. However, about nine minutes later it was noticed that the turbine had low steam seal header pressure due to the steam seal feed valve (S-1) not opening. To protect against turbine damage the turbine vacuum breaker was opened causing a second reactor scram on low condenser vacuum. The stem for the 3-4402-A valve was lubricated and the valve was cycled six times. The amps on this valve initially was ten amps and the final reading was six amps. Upon further pursuing this valve problem, the manufacturer was contacted on this subject. Upon his suggestion work requests have been written to check the torque settings and sealed bearings within this valve to further ensure proper preventative maintenance. This inspection program will be accomplished during the next refueling outage. However, since this reactor scram the 3-4402-A valve has gone through other Circulating Water Flow Reversal evolutions without any problems recurring. The S-1 feed valve manual lever was found engaged and was manually disengaged to allow the S-1 valve to operate normally. Safety significance was minimal. since the reactor scram occurred as intended. This is the first occurrence of this type at Dresden.

January 23, 1985

DJS Ltr #85-74

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Update to Licensee Event Report #84-016-1, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv). This update is submitted to correct the error in the S-1 valve nomenclature.

D.G. Scott

Station Superintendent Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
 File/NRC
 File/Numerical

IE22