

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>SURRY POWER STATION, UNIT 2</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 2 8 1</b>	PAGE (3) <b>1 OF 3</b>
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TITLE (4)  
**STEAM FLOW/FEED FLOW MISMATCH WITH LOW GENERATOR LEVEL**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
<b>1</b>	<b>2</b>	<b>1</b>	<b>8</b>	<b>4</b>	<b>8</b>	<b>4</b>	<b>0</b>	<b>2</b>	<b>0</b>	<b>0</b>	<b>0</b>
<b>1</b>	<b>2</b>	<b>1</b>	<b>8</b>	<b>4</b>	<b>0</b>	<b>0</b>	<b>2</b>	<b>0</b>	<b>0</b>	<b>0</b>	<b>0</b>
<b>0</b>	<b>1</b>	<b>2</b>	<b>9</b>	<b>8</b>	<b>5</b>					<b>0</b>	<b>5</b>
										<b>0</b>	<b>5</b>
										<b>0</b>	<b>0</b>
										<b>0</b>	<b>0</b>

OPERATING MODE (9) <b>N</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) <b>0 2 2</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME <b>R. F. SAUNDERS, STATION MANAGER</b>	TELEPHONE NUMBER AREA CODE: <b>8 0 4</b> <b>3 5 7 - 3 1 8 4</b>
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14) <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH:    DAY:    YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

On December 31, 1984 a unit two startup was in progress with reactor power at 22% and turbine power at 65 megawatts. At 0140 hours while transferring feed flow from the bypass valves to the main feedwater regulating valves, a reactor trip occurred due to low level with steam flow/feed flow mismatch in "B" steam generator.

The transient was caused when operators fed A and C steam generators more than required and reduced steaming. This caused an increase in steaming on 'B' generator and a reduced level.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  <b>SURRY POWER STATION, UNIT 2</b>	DOCKET NUMBER (2)  0   5   0   0   0   2   8   1   8   4   —	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			0   2   0	—   0   0	0   2	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

1. Description of the Event

On December 31, 1984, a unit two startup was in progress with reactor power at 22% and turbine power at 65 Megawatts. Feed flow/Steam flow mismatch signals were locked in on A and B steam generators and A and C steam generators were carrying the majority of the steam load. Feedwater control was in manual and the transition from bypass to main feed regulating valves had commenced on A and C steam generators. At 0140 hours while transferring feed flow from the bypass valves to the main feedwater regulating valves, a reactor trip occurred due to low level with steam flow/feed flow mismatch in "B" steam generator.

2. Safety Consequences and Implications

The purpose of the steam/feedwater flow mismatch coincident with low steam generator level trip is to protect the reactor from a sudden loss of its heat sink. The condensate and feedwater systems were operable during this event to supply feedwater to the generators (Heat Sink remained available). All the protection systems remained functional and the plant parameters remained within the bounds of the accident analysis. Therefore an unreviewed safety question was not created and the health and safety of the public were not affected.

3. Cause

The transient was caused when operators fed A and C steam generators more than required which lowered generator temperature. This caused a reduction in steaming of A and C steam generators and an increased steaming of B steam generator. This, in addition to low feed flow to B steam generator caused the low level which resulted in the unit trip.

4. Immediate Corrective Action

Operators performed all appropriate emergency procedures and function restoration procedures to ensure the plant was returned to a stable condition.

Also, the STA performed the status tree reviews to ensure specific plant parameters were noted and appropriate procedures were used to maintain those parameters within safe bounds.

5. Additional Corrective Action

None.

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		YEAR	SEQUENTIAL NUMBER	DIVISION NUMBER			
		8   4   -	0   2   0   -	0   0	0   3	OF	0   3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

6. Action Taken to Prevent Recurrence

Industry effort is being directed toward the resolution of feedwater control problems at low power levels.

7. Generic Implications

None.

# Vepco

VIRGINIA ELECTRIC AND POWER COMPANY  
Surry Power Station  
P. O. Box 315  
Surry, Virginia 23883

January 29, 1985

Serial No: 84-048

Docket No: 50-281

License No: DPR-32

U. S. Nuclear Regulatory Commission  
Document Control Desk  
016 Phillips Building  
Washington, D.C. 20555

Gentlemen:

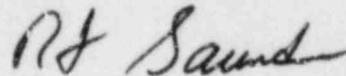
Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

REPORT NUMBER

84-020-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,



R. F. Saunders  
Station Manager

Enclosure

cc: Mr. James P. O'Reilly  
Regional Administrator  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30323

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