UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission or the NPC) has found that:

   A. The application for amendment filed by the Boston Edison Company (the licensee) dated April 8, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 142 , are hereby incorporated in the license The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Victor Nerses

Victor Nerses, Acting Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 15, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove	Insert
155	155

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
3.7.A Primary Containment (Con't)	4.7.A Primary Containment (Con't)
Primary Containment Integrity	Primary Containment Integrity
2.a Primary containment integrity shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel except while performing "open vessel" physics test at power levels not to exceed 5 Mw(t). Primary containment integrity means that the drywell and pressure suppression chamber are intact and that all of the following conditions are satisfied: (1) All manual containment isolation valves on lines connected to the reactor coolant system or containment which are not required to be open during accident conditions are closed. (2) At least one door in each airlock is closed and sealed. (3) All blind flanges and manways are closed. (4) All automatic primary containment isolation valves and all instrument line flow check valves are operable except as specified in 3.7.A.2.b. (5) All containment isolation check valves are operable or at least one containment isolation valve in each line having an inoperable valve is secured in the isolated position.	2.a The primary containment integrity shall be demonstrated by performing Primary Containment Leak Tests in accordance with 10CFR50 Appendix J, with exemptions as approved by the NRC and exceptions as follows: (1) The main steam line isolation valves shall be tested at a pressure $\geq$23 psig, and normalized to a value equivalent to 45 psig. (2) Personnel air lock door seals shall be tested at a pressure $\geq$10 psig. Results shall be normalized to a value equivalent to 45 psig. If the total leakage rates listed below are exceeded, repairs and retests shall be performed to correct the conditions. (1) All double-gasketed seals: 10% L_t (x) (2) All testable penetrations and isolation valves: 60% L_a (x) (3) Any one penetration or isolation valve except main steam line isolation valves: 5% L_t (x) (4) Any one main steam line isolation valve: 11.5 scf/hr @23 psig. where x = 45 psig L_t = .75 L_a L_a = 1.0% by weight of the contained air @ 45 psig for 24 hrs.