



**Wisconsin
Electric**
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VPNPD-92-243
NRC-92-074

10 CFR 50.73

July 9, 1992

U. S. NUCLEAR REGULATORY COMMISSION
Document Control Desk
Mail Station P1-137
Washington, D. C. 20555

Gentlemen:

DOCKETS 50-266 AND 50-301
LICENSEE EVENT REPORT 92-004-00
IMPROPER SEQUENCING OF EMERGENCY SAFETY FEATURES
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Our letter dated June 18, 1992, submitted Licensee Event Report 92-004-00, "Improper Sequencing Of Emergency Safety Features, Point Beach Nuclear Plant, Unit 1." This LER reported the discovery of safeguards load sequencing for both Point Beach Nuclear Plant, Units 1 and 2 being in nonconformance with Technical Specification requirements. Our June 18, 1992, letter and LER incorrectly identified this event as an exclusive Unit 1 event. Therefore, we request that LER 92-004-00 dated June 18, 1992, for Point Beach Nuclear Plant, Unit 1 be withdrawn and replaced with corrected LER 92-004-00, "Improper Sequencing of Safeguards Loads, Point Beach Nuclear Plant, Units 1 and 2," attached.

On May 15, 1992, Operations Refueling Test 3 (ORT-3), "Safety Injection Actuation with Loss of Engineered Safeguards AC," was performed on Unit 1 as required by Technical Specification Section 15.4.6, "Emergency Power System Periodic Tests," Specification A.2. This test is conducted to confirm the ability of the diesel generator to automatically start, shed load, and restore power to particular vital equipment in response to an actual interruption of AC power to the safeguards buses concurrent with a safety injection signal. During initial review and analysis of the test data completed on May 19, 1992, it was identified that the containment ventilation fan and one service water pump sequenced on in greater than the times allowed by Technical Specifications. Subsequent review of data for both the Unit 1 test and the Unit 2 test conducted on November 2, 1991, determined that additional relays did not meet the Technical Specification acceptance criteria.

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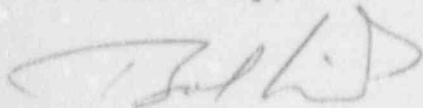
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This report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "The licensee shall report any operation or condition prohibited by the plant's Technical Specification."

If you need any further information, please contact us.

Sincerely,



Bob Link
Vice President
Nuclear Power

Copies to NRC Regional Administrator, Region III
NRC Resident Inspector
Resident Inspector