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C321-92-2211

July 14, 1992

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

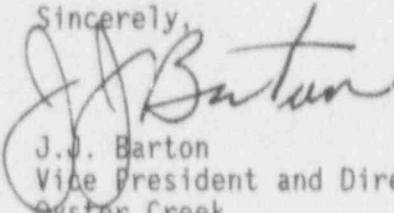
Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Brenda DeMerchant, Oyster Creek Licensing Engineer at (609) 971-4642.

Sincerely,



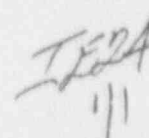
J.J. Barton
Vice President and Director
Oyster Creek

JJB/BDEM: jc
Attachment
(SEP-89) (JUN-92)

cc: Administrator, Region 1
Senior NRC Resident Inspector
Oyster Creek NRC Project Manager

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PDR ADOCK 05000219
R PDR

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Monthly Operating Report
June 1992

Oyster Creek entered June in a Maintenance Outage to perform preventive and corrective actions to enhance plant capability during the summer. On June 7, 1992, the plant began a start-up, but a usually shutdown the following day due to a substantial packing leak on Isolation Condenser Valve V-14-33. After repairing the valve, the plant commenced a start-up and achieved criticality at 2240 on June 10, 1992, but twenty minutes later an automatic scram occurred due to failed IRM bypass switches.

The bypass switches were replaced, and a start-up was again initiated on June 11, 1992. The plant was placed on line at 1728 on June 12, 1992, and continued power ascension until full power was attained on June 14, 1992. The plant remained at full power for the remainder of the month and had a capacity factor of 58.6 %, generating 257,286 megawatts electric net output.

MONTHLY OPERATING REPORT

The following Licensee Event Reports were submitted during the month of June, 1992:

LER 92-005

A reactor scram and subsequent Engineered Safety Features systems actuations were caused by a turbine load rejection due to faults on off-site 230kV transmission lines caused by a forest fire. The scram occurred at 1326 hours on May 3, 1992 and the event concluded at 0635 hours on May 4, 1992. The reactor was operating at approximately 100% power before the scram. Numerous other engineered safety features actuated including Isolation Condensers, Containment Isolation, Diesel Generator fast start, Core Spray and Standby Gas Treatment. Several additional scram signals occurred in the process of bringing the plant to cold shutdown and returning power supplies to off-site sources. An Unusual Event was declared based on high drywell temperature, and an Alert was declared based on the potential of the forest fire to further affect the plant. The plant was brought to cold shutdown at 2234 hours on May 3, and the emergency condition was terminated at 0635 hours on May 4, after off-site power was restored to vital electrical buses. Off-site power had been available since 1331 hours on May 3, but plant management decided not to place the vital buses on off-site power until reliability could be assured. No plant structures or equipment were damaged by the fire. The forest fire which caused the loss of off-site power was the root cause of the event, and the safety significance was minimal because all systems functioned as required. Corrective actions include a revision to the Diesel Generator operating procedure to prevent an avoidable scram when securing diesel generator operation. Utility personnel inspected off-site power lines and found no damage. High resistance contacts on the control rod drive pump time delay relay were replaced due to the pump's failure to start on a diesel generator load sequence.

LER 92-006

On May 10, 1992 while performing an Electromatic Relief Valve (EMRV) Pressure Sensor surveillance, the "As Found" trip setpoint for the high pressure relief function on one EMRV was above that specified in the Technical Specifications. The cause of this occurrence is attributed to setpoint repeatability and instrument drift. The design setpoint repeatability can tolerate instrument drift up to 2.5 psig of the Technical Specification limit. Previous surveillance records indicate that these instruments frequently undergo additional drift within Technical Specification limits due to changing plant and environmental conditions. This occurrence is considered to have minimal safety significance as the automatic depressurization function of the EMRVs is not affected by these pressure switches, all five EMRVs would have actuated to relieve pressure, and the Isolation Condenser System and turbine bypass valves were fully operable. The pressure switch was adjusted to actuate within the Technical Specification limit. A new pressure sensing system is to be installed in accordance with the Oyster Creek Integrated Schedule, which currently specifies the Cycle 15 refueling outage for completion of this project.

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: 06/92
3. UTILITY CONTACT: ED BRADLEY (609)971-4097
4. LICENSED THERMAL POWER (Mwt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 632
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 610
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS:
NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
NONE
11. REASON FOR RESTRICTION, IF ANY:
NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HOURS	720.0	4367.0	197423.0
13. HOURS RX CRITICAL	480.8	4009.3	128372.0
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	918.2
15. HRS GENERATOR ON-LINE	438.5	3952.1	125034.4
16. UT RESERVE SHTDWN HRS	0.0	0.0	1208.6
17. GROSS THERM ENERGY (MWH)	825029	7459544	211784903
18. GROSS ELEC ENERGY (MWH)	269354	2506077	71191707
19. NET ELEC ENERGY (MWH)	257286	2411452	68318140
20. UT SERVICE FACTOR	60.9	90.5	63.3
21. UT AVAIL FACTOR	60.9	90.5	63.9
22. UT CAP FACTOR (MDC NET)	58.6	90.5	55.9
23. UT CAP FACTOR (DER NET)	55.0	85.0	53.2
24. UT FORCED OUTAGE RATE	17.2	4.3	11.3
25. FORCED OUTAGE HRS	91.0	176.8	15868.0

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):

14-R NOVEMBER 27, 1992, 66 - 75 DAYS

27. IF CURRENTLY SHUTDOWN, ESTIMATED STARTUP DATE:

N/A

REFUELING INFORMATION - JUNE, 1992

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: November 27, 1992

Scheduled date for restart following refueling: February 10, 1993

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the NRC.
2. Exxon Fuel Assemblies - No major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core	=	560
(b) in the spent fuel storage pool	=	1708
(c) in dry storage	=	44

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core discharge capacity to the spent fuel pool will be available through the 1996 refueling outage.

AVERAGE DAILY POWER LEVEL
NET MW

DOCKET # 50-219
UNIT. OYSTER CREEK #1
REPORT DATE. JULY 8, 1992
COMPILED BY ED BRADLEY
TELEPHONE # 609-971-4097

MONTH: JUNE, 1992

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	609
2.	0	17.	609
3.	0	18.	609
4.	0	19.	609
5.	0	20.	608
6.	0	21.	618
7.	0	22.	620
8.	0	23.	620
9.	0	24.	649
10.	0	25.	618
11.	0	26.	613
12.	58	27.	594
13.	429	28.	609
14.	533	29.	607
15.	610	30.	606

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219
 UNIT NAME: Oyster Creek
 DATE: July 6, 1992
 COMPLI'D BY: David Egan
 TELEPHONE: 971-4818

REPORT MONTH: June 1992

No.	DATE	TYPE		DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
		F: Forced	S: Scheduled				
119	920530		S	190.5	b	1	Continuation of maintenance outage started on 5/30/92.
120	920608	F		48.5	a	1	Plant was manually shutdown during startup due to a leak on 'B' isolation condenser valve V-14-33.
	920611	F		42.5	a	3	An automatic reactor scram occurred due to faulty IRM bypass switch(es).

SUMMARY:

(1) REASON

- | | |
|--------------------------------|---------------------------------|
| a. Equipment Failure (Explain) | e. Operator Training & Lic Exam |
| b. Maintenance or Test | f. Administrative |
| c. Refueling | g. Operational Error (Explain) |
| d. Regulatory Restriction | h. Other (Explain) |

(2) METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)