PSE&G

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

July 15, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for June are being forwarded to you along with the summary of changes, tests, and experiments for June 1992 persuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan
General Manager -
Hope Creek Operations

RAR
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Attachments

C Distribution

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1/1

The Energy People
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 7/15/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH June 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	790	17.	1041
2.	1036	18.	1042
3.	1041	19.	1037
4.	1049	20.	1040
5.	1040	21.	1015
6.	1040	22.	1052
7.	1014	23.	1049
8.	1026	24.	1031
9.	1038	25.	1048
10.	1041	26.	1039
11.	1058	27.	1036
12.	1037	28.	1034
13.	136	29.	1036
14.	427	30.	1037
15.	979	31.	N/A
16.	1044		

OPERATING DATA REPORT

DOCKET NO. 50-354
UNIT Hope Creek
DATE 7/15/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period June 1992 Gross Hours in Report Period 720

2. Currently Authorized Power Level (MWt) 3293
Max. Depend. Capacity (MWe-Net) 1031
Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	This Month	Yr To Date	Cumulative
5. No. of hours reactor was critical	720.0	4049.5	41,210.8
6. Reactor reserve shutdown hours	0.0	0.0	0.0
7. Hours generator on line	701.1	3987.4	40,562.0
8. Unit reserve shutdown hours	0.0	0.0	0.0
9. Gross thermal energy generated (MWH)	2,231,387	12,768,348	128,765,491
10. Gross electrical energy generated (MWH)	736,650	4,258,530	42,611,024
11. Net electrical energy generated	704,077	4,070,028	40,721,577
12. Reactor service factor	100.0	92.7	85.0
13. Reactor availability factor	100.0	92.7	85.0
14. Unit service factor	97.4	91.3	83.7
15. Unit availability factor	97.4	91.3	83.7
16. Unit capacity factor (using MDC)	94.8	90.4	81.5
17. Unit capacity factor (Using Design MWe)	91.6	87.3	76.7
18. Unit forced outage rate	2.6	3.2	5.0

19. Shutdowns scheduled over next 6 months (type, date, & duration):
Refueling outage, 9/12/92, 60 days

20. If shutdown at end of report period, estimated date of start-up:
N/A

OPERATING DATA REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 7/15/92
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH June 1992

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
5	6/1	F	0	A	4	Power ascension following May forced outage LER 354/92-006
6	6/13	F	18.9	A	9	Unit was taken off line to repair EHC leak. The Reactor was not shut down, but was kept at approximately 3% power for the duration of the outage.

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
UNIT Hope Creek
DATE 7/15/92
COMPLETED BY S. Hollingsworth
TELEPHONE (609) 339-1051

MONTH June 1992

1. Refueling information has changed from last month:

   Yes X No

2. Scheduled date for next refueling: 9/12/92

3. Scheduled date for restart following refueling: 11/11/92

4. A. Will Technical Specification changes or other license amendments be required?

   Yes No X

   B. Has the reload fuel design been reviewed by the Station Operating Review Committee?

   Yes No X

   If no, when is it scheduled? not scheduled (on or prior to 10/28/92)

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

   - Same fresh fuel as current cycle: no new considerations

7. Number of Fuel Assemblies:

A. Incore	764
B. In Spent Fuel Storage (prior to refueling)	760
C. In Spent Fuel Storage (after refueling)	1008

8. Present licensed spent fuel storage capacity: 4006

   Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 11/4, 2010 (EOC16)
   (does not allow for full-core offload)

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

June 1992

On June 1, power ascension was underway following a forced outage on May 26 due to the failure of the Drywell to Suppression Chamber Decay Test to meet its acceptance criteria. The unit reached approximately 100% power on June 1. On June 13, the unit was taken off line to repair an Electro-Hydrualic Control leak. The Reactor remained at approximately 3% power during this evolution, was brought back on line on June 14, and restored to 100% power on June 15. On June 30, the plant had been on line for 16 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

JUNE 1992

The following item has been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluation showed that this item did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. This item did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

DCP Description of Safety Evaluation

TMR	Description of Safety Evaluation
92-016	This TMR installed electrical jumpers across the Feedwater Heater's High High Level Trip Switches. These switches cause spurious high level trip signals during low power levels due to inleakage in the reference leg. The jumpers are only required until the level signals stabilize and were removed later in the month. The Feedwater system is not safety related and is not required to be operable following a LOCA, other than for containment isolation. Failure of the Feedwater system does not compromise any safety related system or components. This TMR has no impact on the containment isolation function of the Feedwater system. Therefore, this TMR does not involve any Unreviewed Safety Questions.