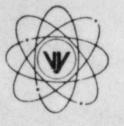
VERMONT YANKEE NUCLEAR POWER CORPORATION



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FVY 85-10

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February 1, 1985

U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Office of Nuclear Reactor Regulation Mr. Domenic B. Vassallo, Chief Operating Reactors Branch No. 2 Division of Licensing

References:

- (a) License No. DPR-28 (Docket No. 50-271)
 - (b) Letter, VYNPC to USNRC, FVY 83-30, dated April 19, 1983
 - (c) Letter, VYNPC to USNRC, FVY 83-85, dated August 4, 1983
 - (d) Letter, VYNPC to USNRC, FVY 83-90, dated August 12, 1983
 - (e) Letter, USNRC to VYNPC, NVY 84-128, dated June 12, 1984, "Issuance of Order Confirming Licensee Commitments on Emergency Response Capability"
 - (f) Letter, VYNPC to USNRC, FVY 84-127, dated October 30, 1984, "NUREG-0737, Supplement I - Regulatory Guide 1.97"
 - (g) EPRI Document NP-3701, dated September 1984 and entitled, "Computer Generated Display System Guidelines"

Dear Sir:

Subject: Safety Parameter Display System

In previous correspondence [References (b), (c), and (d)], Vermont Yankee provided an integrated plan and schedule for addressing the concerns detailed in NUREG-0737, Supplement I, "Requirements For Emergency Response Capability." This overall integrated approach resulted in mutually acceptable program plans for the Emergency Operating Procedures, the Detailed Control Room Design Review, the Regulatory Guide 1.97 Assessment, and a new Emergency Response Facility. Each of these integrated tasks has provided meaningful contributions to the finalization of the Safety Parameter Display System (SPDS) design, the major features of which are described in the enclosed report.

At the present time we have also completed an evaluation of the existing plant computer and its peripheral equipment for suitability to perform the SPDS task. Memory limitations, data trend, expansion, communication, Off-Line calculation, a lack of CRT Compatibility, and approaching computer obsolescense problems have been identified. It is our belief that SPDS and replacement of

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our plant process computer are dependent tasks which will share common components and must proceed concurrently. The existing General Electric supplied Honeywell GEPAC-4020 must be maintained available to support the plant through the next two refueling outages, but will be phased out in a series of equipment upgrades. Our overall schedule is to complete Phase I during the Refueling Outage in 1987 and complete Phase II during the 1988 Refueling Outage. The SPDS installation would be completed concurrent with Phase II.

As committed to in References (c) and (f), the enclosed functional Safety Analysis Report (SAR) provides the Vermont Yankee SPDS design approach, major safety functions and parameters and SPDS implementation schedule. We have elected to follow a design and procurement process similar to that presented in Reference (g) and intend to integrate the SPDS with other related tasks, including NUREG-0737, Supplement I, the replacement of our plant process computer, and future scheduled plant outages. We believe we have established a schedule and program that is both reasonable and consistent with other utilities regarding SPDS and plant process computer replacement.

Should you have any questions on this matter we would be pleased to answer them.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

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Vice President and U Manager of Operations

WPM/ss

Enclosure