

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DIABLO CANYON, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 5	PAGE (3) 1 OF 0 2
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TITLE (4)
TURBINE AUTO STOP OIL PRESSURE-REACTOR TRIP

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
0	1	0 4	8	5	0 0 2	0	1	3 1	DOCKET NUMBER(S) 0 5 0 0 0		
0	1	0 4	8	5	0 0 2	0	1	3 1	DOCKET NUMBER(S) 0 5 0 0 0		

OPERATING MODE (9) **1**

POWER LEVEL (10) **0 1 0 1 8**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.406(c)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 80.38(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 80.38(a)(2)	<input type="checkbox"/> 80.73(a)(2)(vii)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 388A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
DAVID P. SISK, REGULATORY COMPLIANCE ENGINEER	AREA CODE: 8 0 5 NUMBER: 5 9 5 - 7 3 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	B	A F C V	L 2 0 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While in Mode 1 (Power Operation), a reactor trip occurred. All automatic equipment responded as designed.

During plant startup, with the reactor at approximately eight percent power and the main turbine rolling at synchronous speed in preparation for paralleling the main generator to the system, main feedwater pump 1-1 tripped on low lube oil pressure signal. In an attempt to prevent a low-low steam generator level reactor trip, the plant operator tripped the main turbine and reduced reactor power. This resulted in low T_{avg}. Control rods were pulled to restore T_{avg} and reactor power increased to 10 percent, resetting the P-7 permissive. With the P-7 permissive reset, the reactor tripped on turbine auto stop oil pressure. During the event, the turbine driven auxiliary feedwater pump, AFW 1-1, could not be started when inlet valve, FCV-95, did not open on manual demand.

The plant was stabilized in Mode 3 (Hot Standby) in accordance with procedures and all systems and equipment were returned to normal operation. The limit switches for FCV-95 were adjusted. FCV-95 and pump AFW 1-1 were successfully tested and returned to operable status.

This event had no safety consequences and in no way affected the health and safety of the public. This is a previously analyzed Condition II event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1) DIABLO CANYON, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 5 8 5 - 0 0 2 - 0 0	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
					0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1356 PST, January 4, 1985 with Unit 1 in Mode 1 (Power Operation), the reactor tripped on turbine auto stop oil pressure. All automatic equipment responded as designed.

During plant startup, with the reactor (AB) (RCT) at approximately eight percent power and the main turbine (TA) (TRB) rolling at synchronous speed in preparation for paralleling the main generator to the system, main feedwater pump (JB) (P) MFW 1-1 tripped on a low lube oil pressure signal. It is postulated that a voltage transient was caused by the startup of a second condensate booster pump (JB) (P) which momentarily reduced voltage to the lube oil pumps (LL) (P) resulting in lube oil pressure to the feedwater pump turbine (JB) (TRB) 1-1 decreasing below the pump (JB) (P) trip setpoint. In order to prevent a reactor trip, the plant operator manually tripped the main turbine and reduced reactor power to about four percent. Subsequently, while restoring Reactor Coolant System (AB) average temperature, reactor power increased to 10 percent due to control rod withdrawal and Moderator Temperature Coefficient input. Thus P-7, the Lower Power Permissive, reset unblocking the turbine auto stop oil pressure reactor trip. During the event, the turbine driven auxiliary feedwater pump AFW 1-1 could not be started because the pump inlet valve, FCV-95, did not open when activated from the control room.

The plant was stabilized in Mode 3 (Hot Standby) in accordance with procedure and all systems and equipment returned to normal operation. MFW 1-1's lube oil pumps and their controls were tested and no problems were found. Limit switches for FCV-95 were adjusted. FCV-95 and pump AFW 1-1 were successfully tested and returned to operable status approximately six hours after the event.

To reduce the likelihood of similar voltage transients, a Load Flow Study, using actual plant data, has been initiated to provide appropriate recommendations for transformer (EA) (XFMR) tap changes.

This event had no safety consequences and in no way affected the health and safety of the public. This is a previously analyzed Condition II event.

PACIFIC GAS AND ELECTRIC COMPANY

PG&E

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

January 31, 1985

PGandE Letter No.: DCL-85-037

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Diablo Canyon Unit 1
Licensee Event Report 85-002-00
Turbine Auto Stop Oil Pressure - Reactor Trip

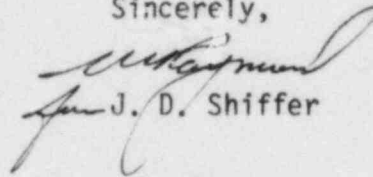
Gentlemen:

Pursuant to 10 CFR 50.73(a)(2)(iv), PGandE is submitting the enclosed Licensee Event Report concerning the inadvertent actuation of an engineered safety feature, reactor trip.

This event has in no way affected the public's health and safety.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,


James D. Shiffer

Enclosure

cc: J. B. Martin
Service List

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