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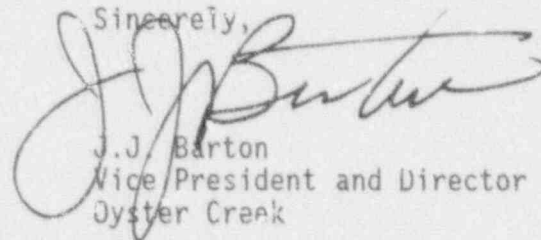
U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report 92-008.

Sincerely,



J.J. Barton
Vice President and Director
Oyster Creek

Enclosure

cc: Administrator, NRC Region 1
Senior NRC Resident Inspector
Oyster Creek NRC Project Manager

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GPU Nuclear Corporation is a subsidiary of General Public Utilities Corporation

Handwritten initials/signature

LICENSEE EVENT REPORT (LER)

APPROVED OMS NO. 3160-0104
EXPIRES - 6/31/85

FACILITY NAME (1) Oyster Creek	DOCKET NUMBER (2) 0 5 0 0 0 2 1 1 9	PAGE (3) 1 OF 3
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TITLE (4)
Technical Specification Shutdown Due to Isolation Condenser Valve Inoperability

EVENT DATE (6)			LER NUMBER (8)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (8)
0 6	0 8	9 2	9 2	0 0 8	0 0	0 7	0 8	9 2			0 5 0 0 0 1 1
0 6	0 8	9 2	9 2	0 0 8	0 0	0 7	0 8	9 2			0 5 0 0 0 1 1

OPERATING MODE (9) **N**

POWER LEVEL (10)

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 72.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 72.71(c)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER: Specify in Abstract below and in Text, NRC Form 365A
<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Paul Cervenka	TELEPHONE NUMBER
	AREA CODE: 6 0 9 TELEPHONE NUMBER: 9 7 1 - 4 8 9 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD'S	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD'S

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	1 2	3 1	9 2

ABSTRACT (Limit to 1400 spaces, i.e., approximately 11 lines single-space typewritten lines) (16)

During a plant startup on June 8, 1992, one of the isolation condensers became inoperable thus requiring a technical specification required shutdown. This condition is considered to be reportable as defined in 10 CFR 50.73(a)(2)(i)(A). The cause of the shutdown was a packing leak on V-14-33 which has been attributed to a scored stem. The cause of the damaged stem is unknown at this time and will be provided in a supplement to this report after the valve is dismantled at some future date. The safety significance of this event is considered minimal since during this period the "A" isolation condenser was available for operation. The stem was stoned and smoothed and the valve was repacked and returned to service.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Oyster Creek	05000219	92	008	00	02	OF	03

DATE OF OCCURRENCE

The event described within this report occurred on June 8, 1992.

IDENTIFICATION OF OCCURRENCE

During a plant startup, one of the isolation condensers became inoperable thus requiring a technical specification required shutdown. This condition is considered to be reportable as defined in 10 CFR 50.73(a)(2)(i)(A).

DESCRIPTION OF OCCURRENCE

On June 8, 1992, at approximately 0535 hours, the control room received a report of a packing leak in the vicinity of the isolation condenser (EIIIS-BL) valves (CFI-ISV). At 0635 hours, it was determined the packing leak was associated with the "B" isolation condenser steam inlet valve V-14-33. At 1400 hours, valve V-14-33 and the associated "B" isolation condenser were declared inoperable while evolutions were in progress to adjust the packing and prepare for the necessary post maintenance testing. This condition required the plant to enter a 30 hour technical specification required shutdown. The adjustment to the packing was unsuccessful in stopping the leak, therefore valve V-14-33 was manually backseated to stop the leakage. The plant shutdown continued and at 1059 hours all control rods were fully inserted. Cold shutdown conditions were reached at 0410 hours on June 9, 1992.

APPARENT CAUSE OF OCCURRENCE

The cause of the shutdown was a packing leak on V-14-33 which has been attributed to a scored stem. This valve is relatively new and was installed during the last refueling outage. The cause of the damaged stem is unknown at this time and will be provided in a supplement to this report after the valve is dismantled at some future date.

ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE

The purpose of the isolation condenser is to depressurize the reactor and remove decay heat without reducing coolant inventory in the event that the main condenser is unavailable as a heat sink.

Technical specifications require two isolation condenser loops to be operable during power operation and any time reactor temperature is above 212 F (except during pressure vessel testing). If one isolation condenser is found to be inoperable during the run mode, the reactor may remain in operation for a period not to exceed seven days provided the motor operated valves in the operable isolation condenser loop are demonstrated daily to be operable. Since the reactor mode switch was still in the startup position and temperature was above 212 F, a reactor shutdown was required within 30 hours.

The safety significance of this event is considered minimal since during this period the "A" isolation condenser was available for operation.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Oyster Creek	DOCKET NUMBER (2) 0 5 0 0 0 2 1 9 3 2	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 2	0 0 8	0 0	0 3	OF	0 3

TEXT (if more space is required, use additional NRC Form 365A (1))

CORRECTIVE ACTION

Immediate corrective action was taken to stone the stem and repack the valve. Subsequently, the valve was backseated to minimize steam leakage. The root cause of the stem scoring is not known. A supplemental report will be submitted when the root cause has been determined.

SIMILAR EVENTS

LER 89-013, "Technical Specification Shutdown due to Isolation Condenser Valve Operator Failure".

FAILURE DATA

Manufacturer: Anchor/Darling
 Type: 900-DD Gate Valve (Double Disk)
 Size: 10 inch