

July 10, 1992

Docket No. 50-446

Mr. William J. Cahill, Jr.  
Group Vice President, Nuclear  
TU Electric Company  
400 North Olive Street, L.B. 81  
Dallas, Texas 75201

Dear Mr. Cahill:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - COMANCHE PEAK STEAM ELECTRIC  
STATION, UNIT 2, PRESSURIZER SURGE LINE LEAK-BEFORE-BREAK ANALYSIS  
(TAC NO. M82947)

By letter dated February 14, 1992, TU Electric requested NRC review and approval of Westinghouse topical report WCAP-13100, "Technical Justification for Eliminating Pressurizer Surge Line Rupture as the Structural Design Basis for Comanche Peak Unit 2." The NRC staff has completed a preliminary review of the submitted report. In order to complete this review, the staff requires additional information as indicated in the enclosure to this letter.

The reporting requirements contained in this letter affect fewer than ten respondents, therefore OMB clearance is not required under Public Law 96-511.

We request your response to the enclosed items within 30 days of the receipt of this letter to enable the staff to complete its review in a timely manner.

Sincerely,

Original Signed By

Brian E. Holian, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosure:  
Request for Additional  
Information

cc w/enclosure:  
See next page

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Mr. William J. Cahill, Jr.

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cc w/enclosure:

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Honorable Dale McPherson  
County Judge  
P. O. Box 851  
Glen Rose, Texas 76043

REQUEST FOR ADDITIONAL INFORMATION

WCAP-13100, "TECHNICAL JUSTIFICATION FOR ELIMINATING PRESSURIZER SURGE  
LINE RUPTURE AS THE STRUCTURAL DESIGN BASIS FOR COMANCHE PEAK UNIT 2"

TU UTILITIES ELECTRIC COMPANY

COMANCHE PEAK STEAM ELECTRIC STATION UNIT 2

DOCKET NO. 50-446

The NRC staff has completed a preliminary review of WCAP-13100. The staff has two concerns: first, the leak flow size for load case B/G shown in Table 7-1 in the submittal differs from the staff's value significantly; second, shielded metal arc weld locations showing lower stresses may become governing locations after taking into account the "Z" factor in the subsequent limit load analysis. To resolve these issues, the staff requests the following additional information:

1. Determination of Leak Flow Size

Supply a copy of actual data input to the computer program and the corresponding computer output for limit load analysis and leakage flow size calculation for Case B/G of Table 7-1.

2. Determination of Governing Locations

Provide a table similar to Table 4-4, which shows the worst stress of all shielded metal arc weld locations along the line for each load case (Cases A through G).