PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE -- PNO-IV-92-30A

This preliminary notification constitutes EAPLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by the Region IV staff on this date.

FACILITY: Fort Calhoun Station

Omaha Public Power District

FORT CALHOUN STATION (FCS)

Docket: 50-285

Licensee Emergency Classification:
Notification of Unusual Event
Alert
Site Area Emergency
General Emergency

Not Applicable

SUBJECT: UPDATE OF PNO-IV-92-30 REGARDING THE DECLARATION OF AN ALERT AT THE

At 11:52 p.m. (CDT) on July 3, 1992, FCS entered an ALERT because of a loss-of-coolant accident that exceeded 40 gallons per minute (gpm).

The licensee has repaired the inverter assembly which failed on July 3, 1992, which resulted in a loss of power to an instrument bus. The nonsafety-related instrument bus provides power to the main turbine's EHC controls and the steam dump valve controls. The initial problem with the inverter appears to have been a weak solder connection on a printed circuit board. This board was replaced on July 3, 1992, however, a metal strap that served as a jumper was not removed from the replaced board and installed on the new board. The failure to install the jumper caused erratic inverter operation when it was reenergized from the battery source. The resultant voltage and current oscillations resulted in tripped circuit breakers and blown fuses. The inverter is presently being load tested by the temporary connection of a load resistor bank.

The licensee has also removed the spilled coolant from the containment sump, which resulted when pressurizer code Safety Valve RS-142 lifted and failed to reseat. Approximately 20,000 gallons of normal reactor coolant activity water was transferred to the liquid radwaste system for processing.

The pressurizer code safety valves have been sent to Wyle Laboratories for testing, inspection, and repair. Preliminary inspection completed at the site indicated that the safety valve which failed to reseat has a skewed valve disk.

Initial inspection at Wyle Laboratories indicates that RS-142 safety relief valve has a ruptured bellows. Testing of the RS-141 safety relief has revealed no anomalies. Valve disassembly will commence today. NRC inspectors from Region IV are following the valve inspection and testing at Wyle Laboratories.

The licensee and NRC have issued press releases and have responded to news media inquiries. A public exit meeting from the Augmented Inspection Team and public meeting to respond to questions will be held at the site today at 10 a.m.

The States of Nebraska and Iowa were informed.

Region IV has informed EDO, NRR, and PA

This information has been confirmed with a licensee representative.

CONTAGT: A. Bill Beach, 817/860-8223

ABBoach:dlf

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