

ENCLOSURE 1

NOTICE OF VIOLATION

Southern Nuclear Operating  
Company, Inc.  
Farley

Docket Nos. 50-348 and 50-364  
License No. NPF-2 and NPF-8

During an NRC inspection conducted on May 18-22, 1992, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C. The violations are listed below:

- A. 10 CFR 50 Appendix B, Criterion III and the Licensee's accepted Operations Quality Assurance Program, FNP-FSAR-17, Section 17.2.3, Design Controls, require that measures shall be established to assure that applicable regulatory requirements and design basis are correctly translated into specifications, drawings, procedures and instructions.

Contrary to the above, on January 10, 1989, Production Change Request No. B-89-1-5586 was approved for replacing installed 96-145 psig range springs with 146-220 psig range springs in various pressure relief valves in the Component Cooling Water and Service Water Systems. Production Change Notice No. B-89-1-5586 was developed for (1) implementing the requested plant modifications and (2) providing design basis information concerning setpoint tolerance for incorporation in the Setpoint Index Drawing No. B-175968. The pressure relief valves springs were replaced in September 1989 and Drawing No. B-175968 was never revised to include the required setpoint tolerances.

This is a Severity Level IV violation (Supplement I).

- B. 10 CFR 50, Appendix B, Criterion III states in part that design control measures shall provide for checking the adequacy of design, such as by the performance of a suitable test program.

During diesel generator (D/G) surveillance testing on October 25, 1991, it was found that when only Unit 2 service water (SW) was aligned to the 1C D/G, adequate SW flow was not achieved for the design basis condition where the SW temperature was greater than 96 degrees Fahrenheit. The required design SW flowrate from Unit 2 only to D/G 1C is 950 gpm. The SW flowrate achieved during the test was 640 gpm. Revision 28 to PCN B87-2-4106, which was implemented during the 1992 Unit 2 refueling

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outage, involved replacing a section of existing carbon steel piping in the return line from the 1C D/G with stainless steel piping.

Contrary to the above, Unit 2 was restarted after the refueling outage and returned to power operation without adequate post modification testing being specified or performed to determine if the required SW design flowrate could be achieved when only Unit 2 SW was aligned to the 1C D/G.

This is Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Southern Nuclear Operating Company, Inc. is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector, Farley, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that will be taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Atlanta, Georgia  
this 26th day of June 1992