

2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS

THERMAL POWER (Low Pressure or Low Flow)

2.1.1 THERMAL POWER shall not exceed 25% of RATED THERMAL POWER with the reactor vessel steam dome pressure less than 800 psia or core flow less than 10% of rated flow.

APPLICABILITY: CONDITIONS 1 and 2.

ACTION:

With THERMAL POWER exceeding 25% of RATED THERMAL POWER and the reactor vessel steam dome pressure less than 800 psia or core flow less than 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours.

THERMAL POWER (High Pressure and High Flow)

1.09

2.1.2 The MINIMUM CRITICAL POWER RATIO (MCPR) shall not be less than ~~1.07~~ with the reactor vessel steam dome pressure greater than 800 psia and core flow greater than 10% of rated flow.

APPLICABILITY: CONDITIONS 1 and 2.

ACTION:

1.09

With MCPR less than ~~1.07~~ and the reactor vessel steam dome pressure greater than 800 psia and core flow greater than 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours.

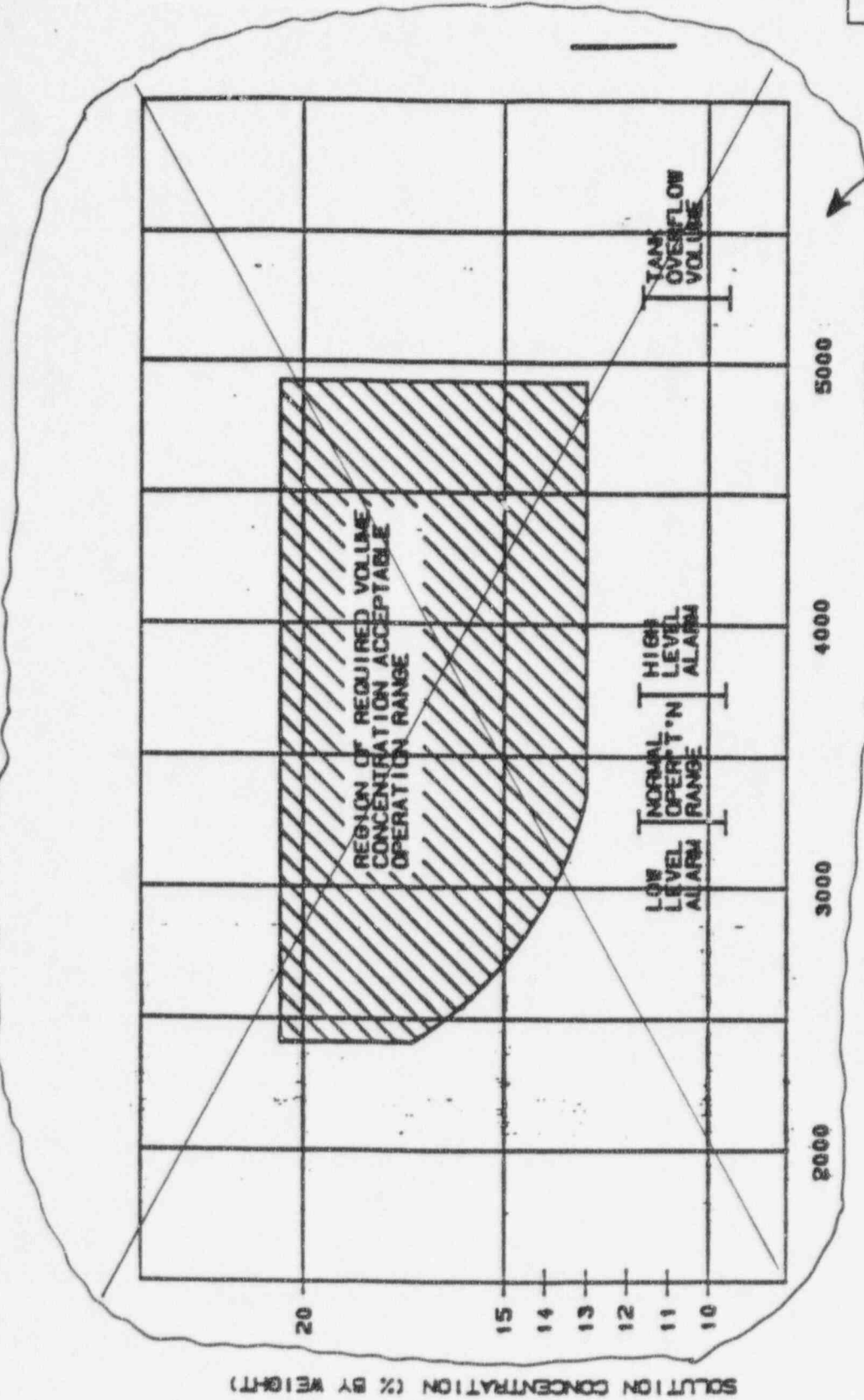
REACTOR COOLANT SYSTEM PRESSURE

2.1.3 The reactor coolant system pressure, as measured in the reactor vessel steam dome, shall not exceed 1325 psig.

APPLICABILITY: CONDITIONS 1, 2, 3, and 4.

ACTION:

With the reactor coolant system pressure, as measured in the reactor vessel steam dome, above 1325 psig, be in at least HOT SHUTDOWN with reactor coolant system pressure \leq 1325 psig within 2 hours.



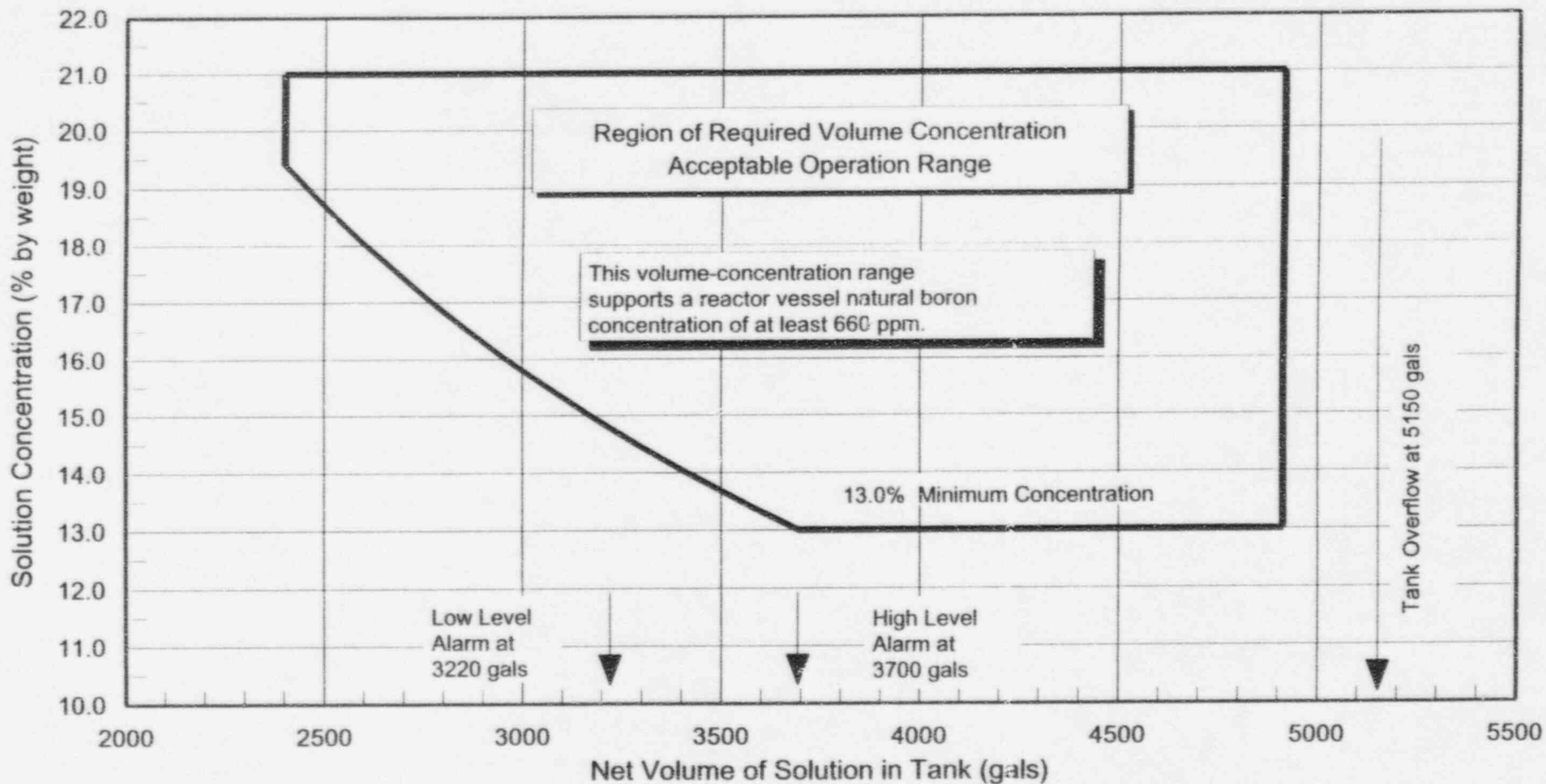
REPLACE AS SHOWN (ATTACHED)

NET VOLUME OF SOLUTION IN TANK (GALS.)
SODIUM PENTABORATE SOLUTION VOLUME CONCENTRATION REQUIREMENTS

FIGURE 3.1.5-1

SODIUM PENTABORATE SOLUTION VOLUME CONCENTRATION REQUIREMENTS

11/15/95



5.0 DESIGN FEATURES

5.1 SITE

EXCLUSION AREA

5.1.1 The exclusion area shall be as shown in Figure 5.1.1-1.

LOW POPULATION ZONE

5.1.2 The low population zone shall be as shown in Figure 5.1.2-1, based on the information given in Section 2.2 of the FSAR.

SITE BOUNDARY

5.1.3 The SITE BOUNDARY shall be as shown in Figure 5.1.3-1. For the purpose of effluent release calculations, the boundary for atmospheric releases is the SITE BOUNDARY and the boundary for liquid releases is the SITE BOUNDARY prior to dilution in the Atlantic Ocean.

5.2 CONTAINMENT

CONFIGURATION

5.2.1 The PRIMARY CONTAINMENT is a steel-lined, reinforced concrete structure composed of a series of vertical right cylinders and truncated cones which form a drywell. This drywell is attached to a suppression chamber through a series of vents. The suppression chamber is a concrete, steel-lined pressure vessel in the shape of a torus. The primary containment has a minimum free air volume of (288,000) cubic feet.

DESIGN TEMPERATURE AND PRESSURE

5.2.2 The primary containment is designed and shall be maintained for:

- a. Maximum internal pressure 62 psig.
- b. Maximum internal temperature: drywell 300°F
suppression chamber 200°F
- c. Maximum external pressure 2 psig.

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 560 fuel assemblies limited to the following fuel types: BP8x8R, GE8x8EB, and GE8x8NB-3, and GE13.

ENCLOSURE 6

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1
NRC DOCKET NO. 50-325
OPERATING LICENSE NO. DPR-71
REQUEST FOR LICENSE AMENDMENT
FUEL CYCLE 11 RELOAD LICENSING

TYPED TECHNICAL SPECIFICATION PAGES - UNIT 1

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APPLICABILITY: CONDITIONS 1 and 2.

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APPLICABILITY: CONDITIONS 1 and 2.

ACTION:

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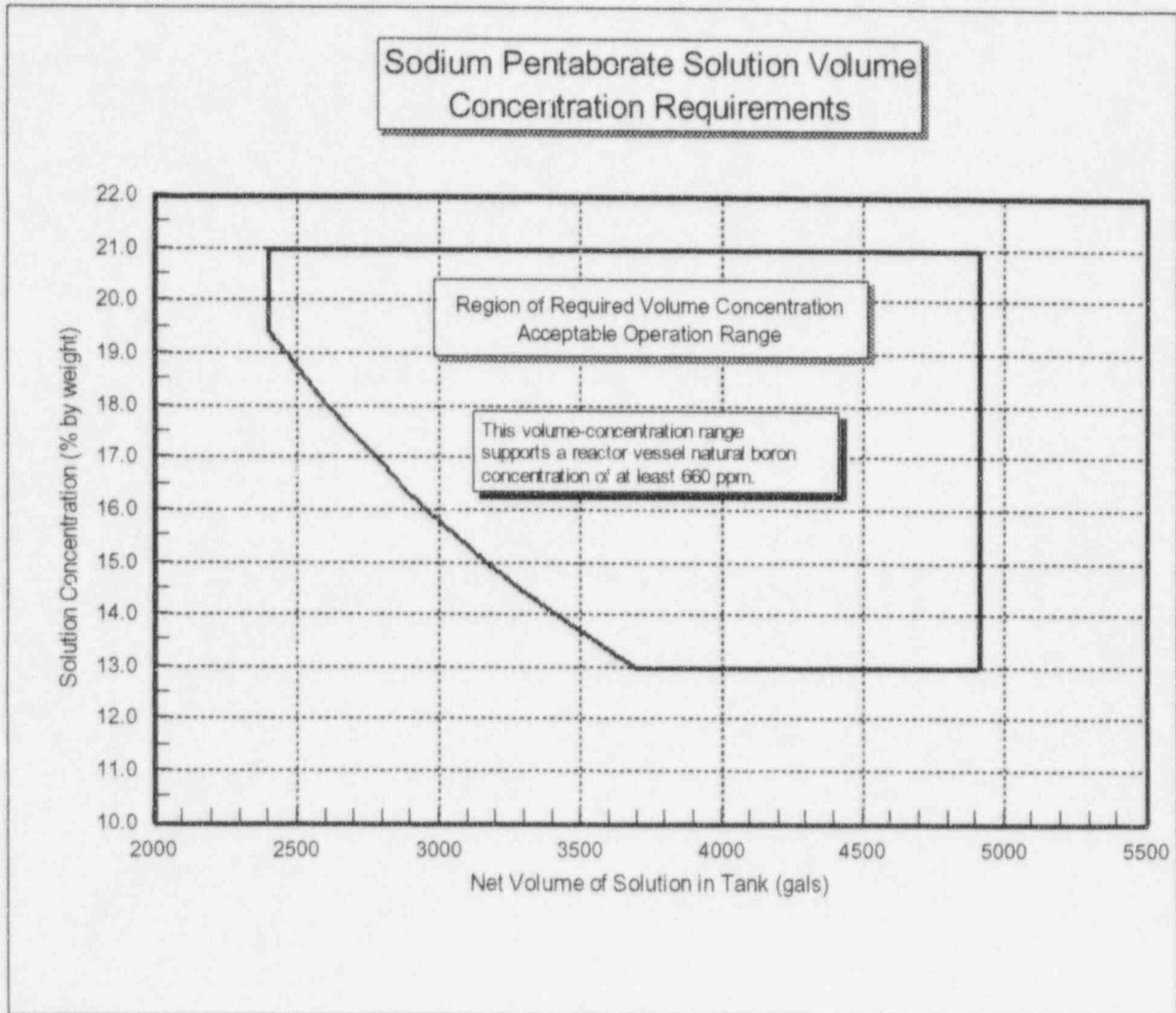
APPLICABILITY: CONDITIONS 1, 2, 3, and 4.

ACTION:

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FIGURE 3.1.5-1

SODIUM PENTABORATE SOLUTION VOLUME
CONCENTRATION REQUIREMENTS



Chronology

- **February 4 - 6, 1996: Action**

- ◆ Toxicologist Review of BNP Pre-Screening Operations
 - No Deficiencies in Set-up or Use of Analysis Equipment
 - Errors Were Mistakes of Transposition and Omission

- **February 14, 1996: Action**

- ◆ Industry Notification Issued Regarding Erroneous Data Reporting During On-Site Vendor Pre-Screening Operations

- **February 19, 1996: Action**

- ◆ Submitted Informational Report on Issues to NRC Staff

CP&L