

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit 1 DOCKET NUMBER (2) 0500002193 PAGE (3) 1 OF 02

TITLE (4) Containment Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
1	2	25	8	4	8	4	0	1	9	0	0	0
											0	5
											0	5

OPERATING MODE (9) N

POWER LEVEL (10) 001

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	<input type="checkbox"/>	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)	<input type="checkbox"/>
20.406(a)(1)(i)	<input type="checkbox"/>	50.38(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)	<input type="checkbox"/>
20.406(a)(1)(ii)	<input type="checkbox"/>	50.38(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vi)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
20.406(a)(1)(iii)	<input type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	<input type="checkbox"/>		
20.406(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	<input type="checkbox"/>		
20.406(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	<input type="checkbox"/>		

LICENSEE CONTACT FOR THIS LER (12)

NAME Richard Schifone - Plant Engineer TELEPHONE NUMBER 617 774 61-719100

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
B	J E	L I C	Y O 1 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 12/25/84, with the reactor at 1% power and the mode switch in startup, a group 1 containment isolation signal was received. The isolation was caused when the "A" reactor water level indicator trended up to 45" and indicated a high reactor water level. The "B" reactor water level remained at 36" during the event. Investigation has shown that the "A" high water level signal was incorrect.

Corrective action will be addressed in Boston Edison's response to Generic Letter 84-23.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Pilgrim Nuclear Power Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 8 4	LER NUMBER (5)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
					0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On 12/25/84, with the plant at 1% power, the reactor coolant temperature at 235°F, the reactor mode switch in start-up and reactor pressure at 10 psig, a group 1 containment isolation signal was received. The signal, which closes the main steam isolation valves and the main steam line drain isolation valves, was generated when the reactor water level indication from the "A" instrumentation trended up to 45". The "B" water level instrumentation remained at 36" throughout the event.

In order to correct the level discrepancy, the reactor water level rack was valved out and a reactor shutdown was initiated in accordance with Technical Specification Table 3.1.1 instrumentation requirements. It has been subsequently determined that the signal generated by the "A" level instrumentation was incorrect. The root cause of the incorrect signal has been determined to be excess cooling in the area of the "A" reference leg which, at low reactor steam temperatures, inhibits the ability of the condensing chamber to provide adequate make-up. The subject level instruments are manufactured by the Yarway Corporation.

To prevent recurrence, proposed modifications to the reactor water level instrumentation are being evaluated. The decision on which modification to implement will be reported in Boston Edison's response to Generic Letter 84-23.

This event did not impact the health and safety of the public.

A previous similar event was reported in LER 82-011.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

January 24, 1985
BECo Ltr. #85-016

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U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

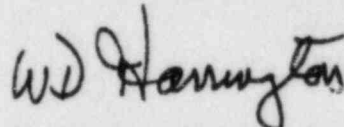
Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 84-019-00, "Containment Isolation," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

RS:caw

Enclosure: LER 84-019-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECo LER Distribution

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