Tech. Spec. 6.9.A.2



Pilgrim Nuclear Power Station Rocky Hill Road Plymouth, Massachusetts 02360

L. J. Olivier Vice President Ruclear Operations and Station Director

April 11, 1996 BECo Ltr. #96-034

U.S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555

> Docket No. 50-293 License No. DPR-35

### MARCH 1996 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.

RLC/dmc/9458

Attachment

cc: Mr. Thomas T. Martin Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

Senior Resident Inspector

150032

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#### **OPERATING DATA REPORT**

DOCKET NO.	50-293
NAME:	Pilgrim
DATE:	April 11,1996
COMPLETED BY:	R. L. Cannon
TELEPHONE:	(508) 830-8321
REPORT MONTH	March, 1996

### OPERATING STATUS

14.14.1	 A	and the second second

- 1. Unit Name
- 2. Reporting Period
- 3. Licensed Thermal Power (MWt)
- 4. Nameplate Rating (Gross MWe)
- 5. Design Electrical Rating (Net MWe)
- 6. Maximum Dependable Capacity (Gross MWe)
- 7. Maximum Dependable Capacity (Net MWe)
- 8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

No Changes

### 9. Power Level To Which Restricted, If Any (Net MWe): None

10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	744.0	2184.0	204336.0
12.	Hours Reactor Critical	744.0	2184.0	128450.1
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14	Hours Generator On-Line	744.0	2184.0	124007.9
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	1486057.0	4258685.0	220560855.0
17.	Gross Electrical Energy Generated(MWH)	512900.0	1469740.0	74747064.0
18.	Net Electrical Energy Generated(MWH)	494961.0	1417614.0	71861236.0
19.	Unit Service Factor	100.0	100.0	60.7
20.	Unit Availability Factor	100.0	100.0	60.7
21.	Unit Capacity Factor (Using MDC Net)	99.3	96.9	52.5
22.	Unit Capacity Factor (Using DER Net)	101.6	99.1	53.7
23.	Unit Forced Outage Rate	0.0	0.0	11.8
24.	Shutdowns Scheduled Over Next 6 Months			
	(Type, Date, and Duration of Each) -	APRI	NED MAINTEN 19, 1996 FOR XXIMATELY SE	ANCE OUTAGE,
and the second second				

25. If Shutdown at End of Report Period, Estimated Date of Startup -

UNIT OPERATING

Pilgrim I March 1996 <u>1998</u> <u>678</u> <u>655</u> <u>696</u> 670

NOTES

# AVERAGE DAILY UNIT POWER LEVEL

			DOCKET NO. 50-293 NAME: Pilgrim DATE: April 11, 1996 COMPLETED BY: R. L. Cannon TELEPHONE: (508) 830-8321 REPORT MONTH March , 1996
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	666	17	665
2	665	18	666
3	666	19	665
4	666	20	666
5	666	21	665
6	666	22	665
7	665	23	665
8	665	24	665
9	666	25	666
10	665	26	665
11	665	27	665
12	665	28	665
13	666	29	665
14	664	30	664
15	665	31	664
16	666		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

#### OPERATIONAL SUMMARY

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REPORT MONTH	March, 1996

The unit started the reporting period at 100 percent core thermal power (CTP) where it was maintained until the end of the reporting period.

#### SAFETY RELIEF VALVE CHALLENGES

#### MONTH OF MARCH 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

#### **REFUELING INFORMATION**

DOCKET NO.	50-293
NAME:	Pilgrim
DATE: Ap	ril 11, 1996
COMPLETED BY:	R. L. Cannon
TELEPHONE:	(508) 830-8321
REPORT MONTH	March, 1996

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

- 1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
- Scheduled date for next refueling shutdown: February 1, 1997.
- Scheduled date for restart following next refueling: March 14, 1997.
- Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
- 5. See #6.
- The new fuel loaded during the 1995 refueling cutage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
- 7. (a) There are 580 fuel assemblies in the core.
  - (b) There are 1765 fuel assemblies in the spent fuel pool.
- 8. (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
  - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
- With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

## PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

DOCKET NO.	50-293
NAME:	Pilgrim
DATE: Ap	ril 11, 1996
COMPLETED BY:	R. L. Cannon**
TELEPHONE:	(508) 830-8321
REPORT MONTH	March, 1996

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
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No major safety related maintenance was completed during this reporting period.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-293	
NAME:	Pilgrim	81
DATE: A	oril 11, 1996	•
COMPLETED BY:	R. L. Cannon	• •
TELEPHONE:	(508) 830-8321	
<b>REPORT MONTH</b>	March, 1996	

NO.	DATE	TYPE	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE	
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There were no unit shutdowns or significant power reductions during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)