



**BOSTON EDISON**

Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

E. Thomas Boulette, PhD  
Vice President Nuclear Operations  
and Station Director

July 14, 1992  
BECO Ltr. #92-076

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

License No. DPR-35  
Docket No. 50-293

Subject: June 1992 Monthly Report

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

E. Thomas Boulette

WJM/bal

Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator, Region 1  
U.S. Nuclear Regulatory Commission  
475 Aliendale Rd.  
King of Prussia, PA 19406

Mr. R. B. Eaton  
Div. of Reactor Projects I/II  
Office of NRR - USNRC  
One White Flint North - Mail Stop 14D1  
11555 Rockville Pike  
Rockville, MD 20852

Senior Resident Inspector

9207130139 920630  
PDR ADDCK 05000293  
R PDR

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	<u>50-293</u>
UNIT	<u>Pilgrim 1</u>
DATE	<u>July 14, 1992</u>
COMPLETED BY	<u>W. Munro</u>
TELEPHONE	<u>(508) 747-8474</u>

MONTH June 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>577</u>	17	<u>663</u>
2	<u>661</u>	18	<u>664</u>
3	<u>664</u>	19	<u>665</u>
4	<u>662</u>	20	<u>664</u>
5	<u>661</u>	21	<u>664</u>
6	<u>656</u>	22	<u>663</u>
7	<u>660</u>	23	<u>665</u>
8	<u>662</u>	24	<u>664</u>
9	<u>660</u>	25	<u>663</u>
10	<u>660</u>	26	<u>664</u>
11	<u>660</u>	27	<u>663</u>
12	<u>661</u>	28	<u>664</u>
13	<u>622</u>	29	<u>664</u>
14	<u>564</u>	30	<u>663</u>
15	<u>666</u>	31	<u>N/A</u>
16	<u>663</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE July 14, 1992  
 COMPLETED BY W. Munro  
 TELEPHONE (508) 747 8474

OPERATING STATUS

Note.

1. Unit Name Pilgrim 1
2. Reporting Period June 1992
3. Licensed Thermal Power (Mwt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 690
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
None
9. Power Level To Which Restricted, If Any (Net MWe) none
10. Reason for Restrictions, if Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>720.0</u>	<u>4367.0</u>	<u>171455.0</u>
12. Number Of Hours Reactor Was Critical	<u>720.0</u>	<u>3989.6</u>	<u>102350.2</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>720.0</u>	<u>3937.5</u>	<u>98440.4</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1420824.0</u>	<u>7707288.0</u>	<u>17141768.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>489930.0</u>	<u>2661530.0</u>	<u>57867944.0</u>
18. Net Electrical Energy Generated (MWH)	<u>471635.0</u>	<u>2562428.0</u>	<u>55613376.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>90.2</u>	<u>57.4</u>
20. Unit Availability Factor	<u>100.0</u>	<u>90.2</u>	<u>57.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>97.8</u>	<u>87.6</u>	<u>48.4</u>
22. Unit Capacity Factor (Using DEK Net)	<u>100.0</u>	<u>89.6</u>	<u>40.5</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>9.8</u>	<u>12.5</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>Midcycle outage - October 1992 Approximately 35 days</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup N/A

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET : 50-293

Operational Summary for June 1992

The unit started the reporting period at approximately 100 percent core thermal power (CTP). On June 1 at 1541 hours while attempting to change out the "B" Recirculation Motor Generator Set outer exciter brushes while on-line, a generator lockout occurred causing the "B" Recirculation Pump to trip. Reactor power dropped to 46 percent CTP. Following change out of the brushes the "B" Recirculation Pump was restarted on June 1 at 1925 hours. Reactor power was returned to 100 percent CTP at 0340 hours on June 2 and maintained until June 13 when at 2100 hours power was reduced to approximately 50 percent to perform a main condenser backwash. Following a successful backwash reactor power was increased to 100 percent at 1412 hours on June 14, where it was essentially maintained for the remainder of the reporting period. Weekly control rod exercises were performed on June 6, 13, 20 and 27.

Safety Relief Valve Challenges  
Month of June 1992

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

## REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number FD-293.
2. Scheduled date for next refueling shutdown: April 1, 1993
3. Scheduled date for restart following next refueling: June 8, 1993
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage and consisted of 168 assemblies.
  7. (a) There are 580 fuel assemblies in the core.
  - (b) There are 1489 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
  - (b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

MONTH June, 1992

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Primary Containment Atmo-spheric Control (PCIS) System	Post Accident Purge and Vent Valve SV-5082B	Failed to indicate "Full open" position during stroke timing test. (PR92.9067)	Limit Switch out of adjustment	Performed Limit Switch adjustment to obtain proper "Full open" position indication. Performed post work test satisfactorily.	None	N/A
Reactor Recirculation System	Recirculation Pump Motor Generator (MG) Set X-204B*	Generator lockout causing "B" Recirculation Pump to trip. (PR92.9075)	Accidental shorting of opposite polarity brush pig-tails on exciter collector rings while changing out MG Set outer exciter brushes on line.	Changed out MG set outer exciter brushes, and restarted the "B" Recirculation Pump.	To be determined	N/A
Reactor Building Closed Cooling Water (RBCCW) System	RBCCW Pump P-201D mechanical seal.	Pump seal leaking	Faulty mechanical seal.	Implemented FRN 92-03-18 to replace original mechanical seal (single) with split seal. (Chesterton type 221)	Replacement of each RBCCW pump mechanical seal on an as needed basis.	N/A

\* Not a Safety System, however, the malfunction caused a power reduction.

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Reactor Core Isolation Cooling (RCIC) System	Terminal block in Junction Box J599.	Weakened termination point on terminal block for Temperature Switch TS-1360-16C.	Repeated loosening and torquing of terminal block screws during quarterly calibrations.	Replaced terminal with installed spare.	Refer to associated LER.	LER 92-007-00. (To be issued)
Salt Service Water (SSW) System	Sample tap on abandoned hypochlorination sample system off the SSW discharge line.	Sheared off pipe nipple. (PR 92-0203.01)	Unknown.	Removed wooden plug and remaining broken nipple; replaced with new nipple and capped using approved code repair. (FRN 92-03-25)	Train personnel to make aware of code repair, and Temp. Mod. requirements.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293  
 NAME PILGRIM 1  
 DATE July 14, 1992  
 COMPILED BY W. MUNRO  
 TELEPHONE 508 747-8474

REPORT MONTH JUNE 1992

NO.	DATE	TYPE1	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT#	SYSTEM CODE4	COMPONENT CODE5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
-----	------	-------	------------------	--------	---------------------------------	-----------------------	--------------	-----------------	-------------------------------------------------

There were no unit shutdowns or significant power reductions during the reporting period required to be reported.

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)