Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

July 14, 1992
LIC-92-219R

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: June 1992 Monthly Operating Report (MOR)

Enclosed is the June 1992 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification Section 5.9.1.

If you should have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Division Manager
Nuclear Operations

WGG/grc

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
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R. P. Mullikin, NRC Senior Resident Inspector
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INPO Records Center
American Nuclear Insurers

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OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	JULY 08 1992
COMPLETED BY	G. R. CAVANAUGH
TELEPHONE	(402) 636-2474

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: JUNE 1992

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
   NA
9. Power Level to which restricted, if any (Net MWe): NA
10. Reasons for restrictions, if any:
    NA

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period..........	720.0	4367.0	164497.0
12. Number of Hours Reactor was Critical	697.3	2168.2	126986.9
13. Reactor Reserve Shutdown Hours......	.0	.0	1309.5
14. Hours Generator On-line............	689.7	2104.9	125482.0
15. Unit Reserve Shutdown Hours.........	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	999520.3	2750171.9	164373897.6
17. Gross Elec. Energy Generated (MWH)..	334823.0	919089.0	54095215.2
18. Net Elec. Energy Generated (MWH)....	318798.0	871756.9	51605500.3
19. Unit Service Factor................	95.8	48.2	76.3
20. Unit Availability Factor...........	95.8	48.2	76.3
21. Unit Capacity Factor (using MDC Net)	92.6	41.8	68.2
22. Unit Capacity Factor (using DER Net)	92.6	41.8	66.4
23. Unit Forced Outage Rate............	4.2	2.3	3.9

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
    FORT CALHOUN STATION IS CURRENTLY IN SHUTDOWN DUE TO A PROBLEM WITH A SAFETY RELIEF VALVE.
25. If shut down at end of report period, estimated date of startup: 07/29/92
26. Units in test status (prior to comm. oper.):

		Forcast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	JULY 08 1992
COMPLETED BY	G. R. CAVANAUGH
TELEPHONE	(402) 636-2474

MONTH JUNE 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	3	17	477
2	53	18	477
3	305	19	478
4	469	20	479
5	482	21	479
6	483	22	479
7	482	23	478
8	482	24	478
9	482	25	475
10	482	26	475
11	481	27	475
12	480	28	478
13	478	29	478
14	478	30	477
15	477	31	
16	477		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE July 8, 1992
COMPLETED BY G. R. Cavanaugh
TELEPHONE (402) 636-2474

REPORT MONTH June 1992

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
92-04	900531	F	30.3	A	1	92-019	RB	CONROD	On May 31, 1992, Control Element Assembly (CEA) No. 35 dropped into the reactor core. Reactor power was reduced to 70% within one hour per Technical Specifications. Attempts to recover the CEA while at 70% power were unsuccessful. The generator was taken off-line early in the morning on June 1, 1992.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June 1992

1. Scheduled date for next refueling shutdown. — September 1993

2. Scheduled date for restart following refueling. — November 1993

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — June 1993

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — New fuel supplier; New LOCA analysis

6. The number of fuel assemblies:

a) in the core	133 Assemblies
b) in the spent fuel pool	529 Assemblies
c) spent fuel pool storage capacity	729 Assemblies
d) planned spent fuel pool storage capacity	Planned to be increased with higher density spent fuel racks.

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by WAWalker Date 7/9/92

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

JUNE 1992
Monthly Operating Report

I. OPERATIONS SUMMARY

Fort Calhoun Station (FCS) was shutdown June 1st and 2nd due to a dropped Control Element Assembly (CEA #35 in Shutdown Group A) caused by a shorted trip coil that occurred on May 31, 1992. The reactor was placed in hot shutdown on June 1, 1992 and the trip coil was replaced. The reactor was taken critical at 0211 on June 2, 1992 and the plant resumed 100% power operation on June 4, 1992.

Both channels of the Qualified Safety Parameter Display System (QSPDS) were declared inoperable on June 4, 1992 due to potential calibration inaccuracies which could have been incurred due to an improperly calibrated piece of test equipment (hand held pyrometer). Both channels were recalibrated using a properly calibrated pyrometer, and declared operable the same day.

Due to severe weather on June 15, 1992, Fort Calhoun Station initiated a Missouri River debris watch and received permission from the State of Nebraska to install deflector logs upstream of the screen house. Excessive debris (mostly hay) caused the Neal Power Station (a fossil plant upstream from FCS) to trip on low condenser vacuum due to plugging of traveling screens. The debris dissipated before reaching FCS.

On June 19, 1992 both control element drive mechanism (CEDM) cooling fans (VA-2A and 2B) tripped due to a false alarm in Containment Fire Detection Zone No. 19. A containment entry was made to ensure no steam leak was occurring near the reactor vessel head (fire was ruled out since other containment fire zones didn't alarm). An emergency temporary modification was initiated to install a jumper into the Zone No. 19 circuitry to allow running VA-2A and 2B.

On June 25, 1992, the linear heat rate (LHR) as displayed on the emergency response facility (ERF) computer reached the administrative limit of 13.75 kW/ft (core operating limit is 13.8 kW/ft). Power was reduced over a three day period from 100% to 98.5% to maintain the LHR at less than 13.75 kW/ft. On June 27, 1992, after completion of a reanalysis of the available margin in LHR based upon actual end of Cycle 13 burnup values (instead of assumed burnup values), the Plant Review Committee (PRC) approved raising the LHR core operating limit to 14.2 kW/ft. The increase in the LHR limit allowed FCS to return the reactor to 100% power.

On June 26, 1992 Turbine Stop Valve No. 2 (SV-2) failed to close during the performance of weekly turbine testing per OI-ST-10. Since SV-2 failed to close, trip unit No. 10 on reactor protection system (RPS) channel "B" was placed in "Bypass". This resulted in FCS entering a 48 hour limiting condition for operation (LCO) per Technical Specification 2.15.1. Troubleshooting revealed that the failure of SV-2 to close was due to a malfunctioning test relay. Upon confirming that the normal valve close circuit was operable, the LCO was exited. A temporary modification completed in early July 1992 installed a switch in place of the test relay. The switch is to be replaced with a new test relay in late July 1992.

On June 30, 1992 the Pyrotronics (TM) XL-3 Fire Detection Panel located in the Control Room caused numerous false alarms due to a failed circuit card. The failure tripped ventilation units in the Control Room and containment, and started both fire pumps. Technical Specification 2.19 required continuous fire watches in Room No. 19 and the other rooms with alarms/malfunctions on the XL-3 Fire Detection Panel. The circuit card was replaced, the XL-3 panel was verified operable and the Technical Specification LCO exited later that same day.

The following NRC Inspections took place during June 1992:

IER No.	Title
91-14	Monthly Resident Inspection

The following LERs were submitted during June 1992:

LER No.	Description
LER-92-014	Reactor Trip Following Maintenance on Moisture Separator Level Instrument
LER-92-018	Corrosion of Boric Acid System Bolts
LER-92-019	Control Element Assembly Drop and Subsequent Plant Shutdown Due to Clutch Coil Failure

A. SAFETY VALVES OR PORV CHALLENGES OR FAILURES THAT OCCURRED

None

B. RESULTS OF LEAK RATE TESTS

Reactor coolant system (RCS) leakage during the month of June 1992 was generally low. Early in June 1992, there were two (2) negative Total and Unknown leak rate test results that occurred during periods of water addition for reactivity control. Also, early in the month, higher than normal leak rates were attributed to charging pump CH-1C packing leakage. This pump was isolated and later repacked beginning on June 13.

Once the plant achieved equilibrium conditions at full licensed power, the Total leak rate was approximately 0.1 gpm. On June 30, the Total leak rate increased to 0.376 gpm due to "B" charging pump discharge valve (CH-192) packing leakage.

C. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
146	This amendment revises the Technical Specifications to increase the maximum allowable setpoint drift for the main steam safety valve (MSSV) setpoint from ±1% to +3%/-2% and to specify lift settings for all MSSVs and for the two pressurizer safety valves.

D. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JUNE 1992

Some of the significant corrective maintenance work performed during the month of June follows:

Lift was adjusted on raw water pump AC-10A.

Spent fuel pool circulating pump AC-5C was rebuilt.

New internal discharge pump check valves and top caps were installed per ECN 90-258 into charging pump CH-1C.

Adjusted relief setpoint on charging pump CH-1C suction relief valve CH-178.

Corrected leak on fuel oil transfer pump suction strainer FO-SA-1.

Replaced instrument air regulator on high pressure safety injection (HPSI) pump SI-2A discharge valve HCV-2928.

Replaced the breaker in MCC-4A2-C04, feeder for boric acid pump CH-4B, to correct tripping problem.

Replaced 52/AS auxiliary switch (General Electric Type SBM) for breaker 1A4-5, feeder for feed pump FW-4C.