1.0 DEFINITIONS (Cont'd)

outage, the required surveillance testing need not be performed until the next regularly scheduled outage.

Reportable Event - A reportable event shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

Run Mode - In this mode the reactor system pressure is at or above 850 psig and the reactor protection system is energized with APRM protection (excluding the 15% high flux trip) and RBM interlocks in service.

Safety Limit - The safety limits are limits below which the reasonable maintenance of the cladding and primary systems are assured. Exceeding such a limit requires unit shutdown and review by the Nuclear Regulatory Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences, but it indicates an operational deficiency subject to regulatory review.

Secondary Containment Integrity - Secondary Containment integrity means that the reactor building is intact and the following conditions are met:

1. At least one door in each access opening is closed.
2. The standby gas treatment is operable.
3. All Reactor Building ventilation system automatic isolation valves are operable or deactivated in the isolation position.

Shutdown - The reactor is in a shutdown condition when the reactor mode switch is in the shutdown mode position and no core alterations are being performed.

Shutdown Mode - Placing the mode switch to the shutdown position initiates a reactor scram and power to the control rod drives is removed. After a short time period (about 10 sec), the scram signal is removed allowing a scram reset and restoring the normal valve lineup in the control rod drive hydraulic system; also, the mainsteam line isolation scram and main condenser low vacuum scram are bypassed if reactor vessel pressure is below 1055 psig.

6.5.2.7 Continued

d. Proposed changes in Technical Specifications or Licenses.

e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.

g. Reportable Event Reports required by 10 CFR 50.73.

h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures,systems, or components.

i. Reports and meeting minutes of the Plant Operation Review Committee.

Audit

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the OSR Committee. These audits shall encompass:

a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.

b. The performance, training and qualifications of the entire facility staff at least once per year.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.

d. The performance of activities required by the Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix B, at least once per two years.

6.6 Reportable Event Action

6.6.1 The following actions shall be taken for Reportable Events:

a. The Commission shall be notified pursuant to the requirements of Section 50.73 to 10 CFR 50.

b. Each Reportable Event Report submitted to the Commission shall be reviewed by the PORC and submitted to the OSR Committee and the Superintendent, Generation Division - Nuclear.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The provisions of 10 CFR 50.36 (c) (1) (i) shall be complied with immediately.

b. The Safety Limit violation shall be reported to the Commission, the Superintendent, Generation Divison - Nuclear or, in his absence, the Superintendent, Generation Division - Fossil-Hydro and to the OSR Committee immediately.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the OSR Committee and the Superintendent, Generation Division - Nuclear within 14 days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet the requirements of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 (November 1972) except as provided in 6.8.2 and 6.8.3 below.

6.9.1 Routine Reports

a. Startup Report - A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be reported in this report.

Startup reports shall be submitted within 90 days following resumption or commencement of commercial full power operation.

b. Annual Occupational Exposure Tabulation (1)

A tabulation shall be made on an annual basis of the number of station utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job function, (2) e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. This tabulation shall be submitted for the previous calendar year prior to March 1 of each year. The dose assignment to various duty functions may be estimated based on pocket dosimeter, TLD, or film badge measurements. Small exposure totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major job functions.

(1) A single submittal may be made for a multiple unit station.

(2) This tabulation supplements the requirements of 10 CFR 20.407.

INTENTIONALLY LEFT BLANK

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of the appropriate Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

a. Loss of shutdown margin, Specification 3.3.A and 4.3.A within 14 days of the event.

b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.

c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.

d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.

e. Calculated dose from release of radioactive effluents, Specification 3.8.B.2, 3.8.B.4, 3.8.C.2, 3.8.C.3, 3.8.C.6, and 3.8.D.

f. Sealed source leakage in excess of limits, Specification 3.13.2.

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

a. Records and logs of facility operation covering time interval at each power level.

b. Records and logs of principle maintenance activities, inspections, repair and replacement of principle items of equipment related to nuclear safety.

c. Reportable Event Reports required by 10 CFR 50.73.

d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.

e. Records of reactor tests and experiments.

f. Records of changes made to Operating Procedures.

g. Records of radioactive shipments.

h. Records of sealed source leak tests and results.

i. Records of annual physical inventory of all source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of facility radiation and contamination surveys.