



Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st. Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

July 10, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for June 1992.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

xc: A. B. Davis, NRC, Region III
D. E. Hills, NRC Resident Inspector LaSalle
J. L. Roman, IL Dept. of Nuclear Safety
B. Siegel, NRR Project Manager
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
D. R. Eggett, NED
P. D. Doverspike, GE Resident
T. J. Kovach, Manager of Nuclear Licensing
T. A. Rieck, Nuclear Fuel Services Manager
J. E. Lockwood, Regulatory Assurance Supervisor
M. M. Servoss, QA/NS Off Site Review
Station File

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LASALLE NUCLEAR POWER STATION
UNIT ,
MONTHLY PERFORMANCE REPORT
JUNE 1992
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 850 Mwe.
	1300	Increased power level to 1105 Mwe.
2	0200	Reduced power level to 900 Mwe to perform monthly surveillances.
	1100	Increased power level to 1100 Mwe.
6	0230	Reduced power level to 1000 Mwe due to system load.
	1100	Increased power level to 1100 Mwe.
7	0000	Reduced power level to 850 Mwe due to system load.
8	1330	Increased power level to 1100 Mwe.
9	0030	Reduced power level to 850 Mwe due to system load.
	1030	Increased power level to 1100 Mwe.
10	0200	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1100 Mwe.
12	0300	Reduced power level to 985 Mwe due to system load.
	0930	Increased power level to 1100 Mwe.
13	2030	Reduced power level to 1015 Mwe due to high lake temperature.
14	0700	Increased power level to 1075 Mwe.
16	0200	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1090 Mwe.
18	0130	Reduced power level to 750 Mwe to withdrawal all control rods, unit in coastdown.
	1100	Increased power level to 1105 Mwe.
19	0230	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1105 Mwe.
20	0100	Reduced power level to 850 Mwe due to system load.

II. MONTHLY REPORT (CONTINUED)

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
	1100	Increased power level to 1105 Mwe.
	1300	Reduced power level to 975 Mwe due to system load.
	1900	Reduced power level to 850 Mwe due to system load.
21	1330	Increased power level to 1055 Mwe.
	1430	Reduced power level to 1000 Mwe to insert a control rod due to APRM high alarm.
22	0030	Reduced power level to 850 Mwe due to system load.
	1400	Increased power level to 1090 Mwe.
23	0030	Reduced power level to 830 Mwe due to system load.
	1100	Increased power level to 1080 Mwe.
24	0230	Reduced power level to 900 Mwe due to system load.
	1000	Increased power level to 1060 Mwe.
	2000	Reduced power level to 850 Mwe due to system load.
	2300	Reduced power level to 775 Mwe due to system load.
25	1100	Increased power level to 1075 Mwe.
26	0130	Reduced power level to 750 Mwe to perform rod set.
	0230	Increased power level to 850 Mwe.
	1100	Increased power level to 1100 Mwe.
28	0130	Reduced power level to 850 Mwe due to system load.
29	1330	Increased power level to 1100 Mwe.
30	0000	Reduced power level to 850 Mwe due to system load, performed monthly surveillances.
	1200	Increased power level to 1100 Mwe.
	2400	Reactor critical, Generator on-line at 850 Mwe, power level reduced due to system load.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-008-00	06/22/92	Spurious instrument spike resulting in a Reactor Core Isolation Cooling system injection and a level 3 'B' channel 1/2 scram.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L13045	Residual Heat Removal High Pressure Alarm Pressure switch 1E12-N018	Instrument stop valve 1E12-F352 leaking.	None.	Replaced stop valve.
L15221	Reactor Core Isolation Cooling Water Leg Pump 1E51-C003	Pump motor.	High motor vibrations.	Replaced motor bearings.
L15743	Hydraulic Control Unit 10-51	Accumulator instrument block cap.	Nitrogen leakage.	Replaced block cap.
L15787	Residual Heat Removal Pump 1E12-C002C	Suction pressure indicator 1E12-R002C.	Inaccurate pressure indication.	Replaced pressure gauge & instrument stop valve.
L15938	Hydraulic Control Unit 06-27	Scram pilot solenoid valves.	None.	Rebuilt solenoid valves.
L15961	Main Steam Leakage Control System Pressure Transmitter 1E32-N061A	Instrument stop valve 1E32-F302A	Instrument stop valve leakage.	Replaced valve.
L16004	Control Room HVAC Ammonia Detector OXY-VC125A	Cassette drive motor out of calibration.	Spurious high level alarms.	Replaced drive motor and recalibrated.
L16202	'1A' Diesel Generator space heater.	Overload relay.	Overload relay failed to trip.	Replaced relay.
L96459	Switchgear Heat Removal Outside Air Supply Damper 1VX14Y	Damper actuator 1TZ-VX009A.	Actuator failed to cycle.	Replaced actuator.

(No SOR Failures this month.)

ZCADTS/5+

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE July 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: June 1992
GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	720.0	4,305.5	51,662.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	720.0	4,272.8	50,651.1
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	2,266,533	13,726,170.8	148,883,150
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	745,059	4,650,003	49,739,775
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	719,010	4,502,929	47,698,074
12. REACTOR SERVICE FACTOR (%)	100.0	98.6	69.3
13. REACTOR AVAILABILITY FACTOR (%)	100.0	98.6	71.5
14. UNIT SERVICE FACTOR (%)	100.0	97.8	68.0
15. UNIT AVAILABILITY FACTOR (%)	100.0	97.8	68.0
16. UNIT CAPACITY FACTOR (USING MDC) (%)	96.4	99.5	61.8
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	92.6	95.7	59.4
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	2.2	7.2

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
Refuel Outage, 09/26/92, 12 Weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE July 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: June 1992

DAY	POWER	DAY	POWER
1	1,003	17	1,050
2	1,022	18	1,025
3	1,073	19	1,023
4	1,077	20	907
5	1,074	21	919
6	1,037	22	953
7	830	23	975
8	982	24	947
9	994	25	949
10	1,011	26	1,000
11	1,068	27	1,063
12	1,051	28	831
13	1,036	29	982
14	1,041	30	982
15	1,043	31	
16	1,012		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
2	92G607	F	0.0	H	4	Power reduction due to system load.
3	920628	F	0.0	H	4	Power reduction due to system load.

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations
(None.)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECC System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0497	000004	Adjust discharge valve limit switches.
0502	000004	Inspect transfer pump make up valve.
(U-1)		
0470 0471 0482	1E12-F026B	Inspection.
0472 0473 0483	1E12-F003B	Inspection.
0476	1DG01K	Rewire annunciator logic.
0480	1DG01K	Inspection main feed breaker.
0486	1DG01K	Change soakback oil filter.
0493	1DG01K	Replace soakback pump control power transformer.
0505	1E12-F352	Replace instrument stop valve.
0516	1E51-F046	Perform Votes test.
0526	1E51-C002	Administrative control.

LASALLE NUCLEAR POWER STATION
UNIT 2
MONTHLY PERFORMANCE REPORT
JUNE 1992
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1115 Mwe (power level held due to high vibrations on the #11 Main Turbine bearing).
2	2200	Increased power level to 1125 Mwe.
7	0030	Reduced power level to 1115 Mwe due to high vibrations on the Main Turbine.
8	0600	Increased power level to 1125 Mwe.
10	0030	Reduced power level to 850 Mwe to perform monthly and quarterly surveillances.
	1000	Increased power level to 1125 Mwe.
15	0300	Reduced power level to 1050 Mwe due to system load.
	0600	Increased power level to 1115 Mwe.
16	0100	Reduced power level to 950 Mwe due to system load.
	1100	Increased power level to 1120 Mwe.
17	0230	Reduced power level to 985 Mwe due to the 231A and 231B switchgears tripping off-line.
	1000	Increased power level to 1125 Mwe.
18	0130	Reduced power level to 850 Mwe due to system load.
	1100	Increased power level to 1120 Mwe.
20	0100	Reduced power level to 1050 Mwe due to system load.
21	0100	Reduced power level to 830 Mwe due to system load.
	1300	Increased power level to 1095 Mwe.
22	0100	Reduced power level to 850 Mwe due to system load.
	1400	Increased power level to 1095 Mwe.
23	0100	Reduced power level to 850 Mwe due to system load.
	0900	Increased power level to 1095 Mwe.
24	0300	Reduced power level to 940 Mwe due to system load.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

<u>Day</u>	<u>Time</u>	<u>Event</u>
	0800	Increased power level to 1075 Mwe.
25	0000	Reduced power level to 850 Mwe due to system load.
	0900	Increased power level to 1095 Mwe.
	2000	Reduced power level to 835 Mwe due to system load, placed the '2B' Turbine Driven Reactor Feed Pump off-line.
26	1000	Increased power level to 1020 Mwe.
27	0030	Reduced power level to 850 Mwe due to system load.
	1130	Increased power level to 1000 Mwe.
28	0400	Reduced power level to 850 Mwe due to system load.
29	1600	Increased power level to 1120 Mwe.
30	0200	Reduced power level to 850 Mwe due to system load.
	1300	Increased power level to 1120 Mwe.
	2400	Reactor critical, Generator on-line at 1115 Mwe, power level reduced to due system load.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-007-00	06/13/92	Automatic startup of the 'B' Control Room/Auxiliary Electric Equipment Room HVAC Emergency Make Up Train.
92-008-00	06/15/92	During performance of an inservice test, the breaker for valve 2E51-F086 tripped on thermal overload.

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L13879	Control Room HVAC Damper OVC11YB	Damper actuator OFZ-VC047A.	Actuator failed to open damper.	Replaced actuator.
L15084	Reactor Core Isolation 74 relay. Cooling Isolation Valve 1ES1-F019		Inaccurate alarm.	Replaced relay.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE July 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	June 1992		
GROSS HOURS IN REPORTING PERIOD:	720		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	720.0	2,017.2	46,953.1
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	720.0	1,855.4	46,059.9
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWhT)	2,232,095.0	5,456,772	137,699,171
10. ELECTRICAL ENERGY GENERATED (MWhE-Gross)	764,700.0	1,858,324	45,748,074
11. ELECTRICAL ENERGY GENERATED (MWhE-Net)	738,424	1,764,992	43,896,496
12. REACTOR SERVICE FACTOR (%)	100.0	46.2	69.5
13. REACTOR AVAILABILITY FACTOR (%)	100.0	46.2	72.1
14. UNIT SERVICE FACTOR (%)	100.0	42.5	68.2
15. UNIT AVAILABILITY FACTOR (%)	100.0	42.5	68.2
16. UNIT CAPACITY FACTOR (USING MDC) (%)	99.0	39.0	62.8
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	95.1	37.5	60.3
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	5.2	12.8
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			



TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE July 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: June 1992

DAY	POWER	DAY	POWER
1	1,069	17	1,058
2	1,069	18	1,000
3	1,084	19	1,056
4	1,084	20	1,004
5	1,085	21	950
6	1,082	22	956
7	1,080	23	972
8	1,082	24	1,006
9	1,075	25	917
10	1,020	26	930
11	1,081	27	918
12	1,079	28	841
13	1,074	29	992
14	1,074	30	994
15	1,073	31	
16	1,063		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

<u>YEARLY</u> <u>SEQUENTIAL DATE</u> <u>NUMBER</u> (YYMMDD)	<u>TYPE</u> F: FORCED S: SCHEDULED	<u>DURATION</u> (HOURS)	<u>REA[^]ON</u>	<u>METHOD OF</u> <u>SHUTTING DOWN</u> <u>THE REACTOR OR</u> <u>REDUCING POWER</u>	<u>CORRECTIVE</u> <u>ACTIONS/COMMENTS</u> (LER/DVR # if applicable)
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(None.)

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations
(None.)

2. ECCS System Outages
(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.
(None.)

4. Major changes to Radioactive Waste Treatment Systems.
(None.)

5. Indications of Failed Fuel Elements.
(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1796 1810	2E22-S001	Remove and repair thermocouple.
1814	2DG09DB	Air dryer calibrations.
1820	2E51-C004	Pump motor inspection.
1821	2E51-F019	Replace 74 relay.
1828	2E51-F080	Administrative control.
1851 1852	2E12-F411B	Inspection, adjust limit switches.
1853	2DG08CA	Air compressor suction and discharge valve inspection.
1856	2DG08CB	Air compressor suction and discharge valve inspection.
1859	2DG01K	Rewire annunciator logic.