

following DCAs were involved: DCA 8771, 8996, 9602, 9934, 10266, 10267, 10450, 10761, 12525, 12635, 12667, 12668, and 12783. The design errors in the ranges and/or setpoints of the instruments represents an apparent deficiency in the final design and a breakdown in the QA program in not identifying the problems. If the incorrectly aligned instruments had been used and the situation gone uncorrected, it could have adversely affected safe operation of the nuclear plant.

Failure to report these deficiencies to the NRC in accordance with 10 CFR 50.55(e) is a violation (445/8434-02).

5. Inspection and Enforcement Bulletin (IEB) Followup

The NRC inspectors reviewed the licensee's file for each of the IEBs discussed below and performed inspections when required to verify that the licensee had conducted an adequate review to determine if the IEB was applicable to the CPSES facility, and to verify that the licensee had taken the required action on applicable IEBs.

a. IEB 79-15: Deep Draft Pump Deficiencies

This bulletin was applicable to the four service water pumps used at CPSES. The Unit 1 service water pumps have been in operation since March 1981, and have not required any significant repairs. The Unit 1 pumps have been tested as a part of the service water system in accordance with Preoperational Test Procedure 1-CP-PT-04-01, Revision 1, "Station Service Water." The licensee has instituted procedural requirements that accomplish operation, testing, repair, and inservice inspection of these pumps. These procedures were:

OPT-207A, Revision 0, "Service Water System Operability"

MMI-310, Revision 0, "Service Water Pump Inspection"

No further questions are required of this matter at this time.

b. IEB 83-08: Electrical Circuit Breakers With an Undervoltage Trip Feature in Use in Safety-Related Applications Other Than the Reactor Trip System.

The licensee's response letter TXX 4136, dated April 2, 1984, stated that a review of CPSES electrical systems determined that no circuit breakers (other than those used for reactor trip purposes) provided with the "undervoltage trip" feature were used or were planned for use at CPSES.

No further questions are required of this matter at this time.

6. Onsite Followup of Safety Evaluation Report (SER) Open Items

The NRC inspectors held discussions with cognizant licensee personnel, and reviewed related documentation to verify that the licensee had fulfilled commitments made to the SER items discussed below:

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