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Quad Cities Nuclear Power Station
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GCT-92-29

July 2, 1992

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of June 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

Gerald C. Tietz

Gerald C. Tietz
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JUNE 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One began the month of June in a maintenance outage for tube repair on the main condenser. The unit was brought back on line on the 2nd of June. Another maintenance outage, occurred from June 13 through June 15, for additional condenser tube repair.

Full power for the Unit One declined over the month due to the unit being in coast down for the upcoming refuel outage.

The significant load reductions ordered by Chicago Load Dispatch occurred on; June 5 to 630 MWe, June 7 to 600 MWe, June 8 to 700 MWe, and June 9 to 500 MWe.

B. Unit Two

Quad Cities Unit Two performed a load drop per Chicago Load Dispatch on June 4 to 500 MWe. On June 6, Unit Two reduced power to 450 MWe and held due to increased off gas flow. Power, on June 6, was returned to full and reduced a second time to 450 MWe due to increasing condenser back pressure. Unit Two's power remained between 450 MWe to as low as 180 MWe, due to low condenser back pressure and high off gas flow, until condenser leak repairs were complete on June 14.

Chicago Load Dispatch requested the following additional load drops for Unit Two for the month of June;

Date	Load
06-18-92	468 MWe
06-19-92	550 MWe
06-20-92	700 MWe
06-21-92	250 MWe
06-23-92	650 MWe
06-25-92	700 MWe
06-28-92	550 MWe
06-29-92	700 MWe

On June 27 Unit Two generator output was reduced to 700 MWe for a turbine surveillance.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q01006	2406	Repair Oxygen-Hydrogen recorder failed up-scale.	As Found: Recorder pegged high when turned on. As Left: The recorder read in normal range after being turned on for 1 1/2 hours. Solution was sufficient time to stabilize after energizing record panel.
Q01180	1705	MSL rad monitor down scale reactor building fuel pool vent rad monitor calibration bypass switch trouble.	As Found: Bypassed the 1 Rx Bldg vent monitor and MSL downscale alarm came up with no indication on monitors. As Left: Suspected short between leads required lifting and jumping various leads to isolate. No short found. Problem disappeared with no MSL down scale alarm w/process rad monitor in bypass.
Q01274	0312	SCRAM inlet valve leaking water, repair/repack.	As Found: Valve packing was leaking. As Left: Installed new packing and set seating pressure and spring tension.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q01292	2499	Maintenance for valve cam check which failed LLRT.	As Found: Valve cover appeared to not be on all the way. As Left: Gasket had been installed wrong. New gasket installed and cover tightened.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
92-013	06-09-92	Logic deficiency in the SBGT Logic.
92-016	06-26-92	Missed Tech Spec Surveillance on Fire Valve.

UNIT 2

92-018	05-29-92	Manual Isolation of HPCI System.
92-019	06-06-92	Hi Rad Area door lock malfunction (Hotwell Door).
92-020	06-12-92	"R" Gate Found Open.
92-021	06-23-92	Offgas Isolation.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE July 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 060192 / 2400 063092 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
   DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	644.10	3971.60	140482.70
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	622.10	3931.20	136162.30
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	1421217.60	9437121.60	293504144.60
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	450504.0	3062333.0	95136669.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	428921.0	2954096.0	89664264.0
12. REACTOR SERVICE FACTOR	89.45	90.95	79.26
13. REACTOR AVAILABILITY FACTOR	89.46	90.95	81.19
14. UNIT SERVICE FACTOR	86.40	90.02	76.82
15. UNIT AVAILABILITY FACTOR	86.40	90.02	77.34
16. UNIT CAPACITY FACTOR (Using MDC)	77.47	87.99	65.79
17. UNIT CAPACITY FACTOR (Using Design MWe)	75.50	85.76	64.12
18. UNIT FORCED OUTAGE RATE	13.60	10.00	5.85

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE July 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 2400 060192 063092 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	1275.55	134758.25
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2985.8
7. HOURS GENERATOR ON LINE	720.00	1204.55	131224.45
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	1373172.0	2381116.80	282465002.60
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	445726.00	765827.00	90696017.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	424231.00	706141.00	85840316.00
12. REACTOR SERVICE FACTOR	100.00	29.21	76.71
13. REACTOR AVAILABILITY FACTOR	100.00	29.21	78.41
14. UNIT SERVICE FACTOR	100.00	27.58	74.70
15. UNIT AVAILABILITY FACTOR	100.00	27.58	75.10
16. UNIT CAPACITY FACTOR (Using MDC)	76.62	21.03	63.54
17. UNIT CAPACITY FACTOR (Using Design MWe)	74.68	20.49	61.93
18. UNIT FORCED OUTAGE RATE	0.0	0.0	6.02

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE July 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH June 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	- 8	17.	744
2.	101	18.	736
3.	647	19.	731
4.	764	20.	734
5.	729	21.	727
6.	768	22.	709
7.	726	23.	707
8.	749	24.	701
9.	695	25.	705
10.	756	26.	700
11.	744	27.	652
12.	695	28.	657
13.	7	29.	679
14.	- 8	30.	651
15.	39	31.	
16.	62?		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE July 2, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH June 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	784	17.	777
2.	785	18.	701
3.	771	19.	726
4.	663	20.	735
5.	776	21.	539
6.	466	22.	718
7.	367	23.	761
8.	310	24.	776
9.	270	25.	754
10.	330	26.	779
11.	153	27.	742
12.	88	28.	717
13.	102	29.	756
14.	121	30.	780
15.	642	31.	
16.	771		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit One

COMPLETED BY Matthew Benson

DATE July 2, 1992

REPORT MONTH June, 1992

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-23	6-1-92	F	35.6	B	4	- - - -	- -	- - - - -	Continued Cond. Tube Repair
92-24	6-5-92	S	4.5	F	5	- - - -	- -	- - - - -	Load Drop per Chicago Load Dispatcher
92-25	6-7-92	S	4.6	F	5	- - - -	- -	- - - - -	Load Drop per Chicago Load Dispatcher
92-26	6-8-82	S	2.8	F	5	- - - -	- -	- - - - -	Load Drop per Chicago Load Dispatcher
92-27	6-9-92	S	5.9	F	5	- - - -	- -	- - - - -	Load Drop per Chicago Load Dispatcher
92-28	6-13-92	F	62.3	B	2	- - - -	- -	- - - - -	Cond. Tube Repair

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit Two

COMPLETED BY Matthew Benson

DATE July 2, 1992

REPORT MONTH June, 1992

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-05	6-04-92	S	17.1	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-06	6-06-92	F	8.6	D	5	- - - -	- -	- - - -	Increased Off Gas/Cond Leak
92-07	6-06-92	F	202.0	D	5	- - - -	- -	- - - -	Increased Off Gas/Cond Leak
92-08	6-18-92	S	6.9	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-09	6-19-92	S	4.8	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-10	6-20-92	S	6.6	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-11	6-21-92	S	35.0	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-12	6-23-92	S	2.9	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-13	6-25-92	S	5.0	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-14	6-27-92	S	7.6	B	5	- - - -	- -	- - - -	Turbine Weekly Test
92-15	6-28-92	S	6.3	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher
92-16	6-29-92	S	4.3	F	5	- - - -	- -	- - - -	Load Drop per Chicago Load Dispatcher

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 1-1-92 TO 12-31-92

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
2-19-92	2	0.28	0.67	1.43	2.48	J-2 2.55	Drive Replacement (J-2), Scram Valve N-7
2-20-92	1	0.32	0.69	1.45	2.45	N-5 2.45	Scram Valve Work N-5
5-12-92	177	0.31	0.69	1.47	2.58	L-13 3.43	Start-up Scram Timing Unit Two
6-02-92	1	0.31	0.65	1.37	2.35	N-5 2.35	Scram Valve Work
6-26-92	2	0.27	0.62	1.32	2.32	C-13 2.26	ACCUM C-13, N-7

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 9-5-92

3. Scheduled date for restart following refueling: 12-5-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   1. Modification to HPCI turbine exhaust steam line.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   1. 06/30/92

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1405

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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O.C.O.S.R.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 03/06/93

3. Scheduled date for restart following refueling: 06/05/93

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2439

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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OCT 30 1989
Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Friction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center