

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0108
EXPIRES - 6/30/95

FACILITY NAME (1) **Limerick Generating Station - Unit 1** DOCKET NUMBER (2) **0 5 0 0 0 3 1 5 1 2 1** OF **0 1 2**

TITLE (4) **Reactor Enclosure HVAC Isolation**

EVENT DATE (8)			LER NUMBER (8)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (8)
11	21	84	84	0114		11	22	84		0 5 0 0 0 1 1 1
										0 5 0 0 0 1 1 1

OPERATING MODE (6) _____

POWER LEVEL (8) _____

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

<input type="checkbox"/>	20.402(a)	<input type="checkbox"/>	20.406(a)	<input checked="" type="checkbox"/>	60.73(a)(2)(ii)	<input type="checkbox"/>	73.71(a)
<input type="checkbox"/>	20.406(a)(1)(ii)	<input type="checkbox"/>	60.34(a)(1)	<input type="checkbox"/>	60.73(a)(2)(iv)	<input type="checkbox"/>	73.71(a)
<input type="checkbox"/>	20.406(a)(1)(iii)	<input type="checkbox"/>	60.34(a)(2)	<input type="checkbox"/>	60.73(a)(2)(v)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/>	20.406(a)(1)(iv)	<input type="checkbox"/>	60.73(a)(2)(ii)	<input type="checkbox"/>	60.73(a)(2)(viii)(A)	<input type="checkbox"/>	
<input type="checkbox"/>	20.406(a)(1)(v)	<input type="checkbox"/>	60.73(a)(2)(iv)	<input type="checkbox"/>	60.73(a)(2)(viii)(B)	<input type="checkbox"/>	
<input type="checkbox"/>	20.406(a)(1)(vi)	<input type="checkbox"/>	60.73(a)(2)(ii)	<input type="checkbox"/>	60.73(a)(2)(ix)	<input type="checkbox"/>	

LICENSEE CONTACT FOR THIS LER (12)

NAME **B. L. Clark, Senior Engineer-Special Projects** TELEPHONE NUMBER **2 1 5 8 4 4 - 5 0 1 4 7**

AREA CODE _____

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	V, A			N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (16)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single space typewritten lines) (18)

Abstract: 84-014

In order to verify that instrument power was available to the reactor enclosure plant heating instrumentation during system troubleshooting on November 21, 1984, an operator de-energized and then re-energized the electrical circuit which provides power to temperature switches and a temperature transmitter for the plant heating steam system flash tank. When the circuit was de-energized the reactor enclosure heating, ventilating, and air conditioning (HVAC) system isolated as a result of de-energizing relays in the ventilation control circuit. The relays which lost power normally are de-energized by closure of the secondary containment isolation dampers, which then trip the system exhaust fans. The reactor enclosure then isolated due to low enclosure differential pressure. The reactor enclosure HVAC system was restored to operable status after the circuit was re-energized.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2	LER NUMBER (3)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	- 0 1 4	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A (1))

Description of the Event:

On November 21, 1984 at 10:25 p.m., during troubleshooting of the reactor enclosure plant heating system, an operator de-energized and then re-energized circuit 36 in electrical panel 10Y201 to verify that power was available to the reactor enclosure plant heating instrumentation. This circuit supplies power to temperature switches and a temperature transmitter for the plant heating steam flask tank. Circuit 36 also supplies power to the reactor enclosure HVAC panel 10C205. When the circuit was de-energized the 'A' reactor enclosure equipment compartment exhaust fan and the two running reactor enclosure exhaust fans tripped resulting in an isolation of the reactor enclosure HVAC system from low enclosure differential pressure.

Consequences of the Event:

Reactor enclosure HVAC system operability is required in the power operation, startup or hot shutdown modes of operation. Since initial criticality has not yet been achieved, there are no adverse consequences of this event.

Cause of the Event:

The cause of this event is the failure of the operator to properly investigate equipment controlled by the feed switch prior to de-energizing the circuit.

Corrective Actions:

Each shift will be counseled to thoroughly investigate applicable drawings prior to cycling feed switches to determine the effect on other plant equipment.

Previous Similar Occurrence:

LER 84-006

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

December 21, 1984

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

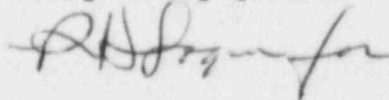
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This Licensee Event Report concerns isolation of the reactor enclosure heating, ventilation, and air conditioning system prior to initial criticality.

Reference: Docket No. 352
Report Number: 84-014
Revision Number: 00
Event Date: November 21, 1984
Report Date: December 21, 1984
Facility: Limerick Generating Station
P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10CFR50.73 (a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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