LICENSEE EVENT REPORT

		. EIGENGEE EVENT HET ONT
		CONTROL BLOCK: [] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
	0 1	0 H D B S 1 2 Ø Ø - Ø Ø Ø Ø - Ø Ø 3 4 1 1 1 1 4 5 5 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58
1	ON'T 0 1 7 8	REPORT L 6 0 5 0 0 0 3 4 6 7 0 6 2 5 8 1 8 1 9 75 REPORT DATE 80
	0 2	[(NP-33-81-45) On 6/25/81 during a routine recovery following a reactor trip, a control
	0 3	room operator discovered that control rod (CR) 5-8 was not moving while withdrawing
	0 4	group 5. CR 5-8 was declared inoperable at 0530 hours, placing the unit in action
	0 5	statement (a) of TS 3.1.3.1. There was no danger to the health and safety of the
	0 6	public or station personnel. The unit was in the hot standby mode at the time of the
	0 7	event, and the existence of an adequate shutdown margin was verified.
	0 8	1 80
	7 8	SYSTEM CAUSE CAUSE SUBCODE SUB
4		TO REPORT NUMBER NUMBER 21 22 23 24 26 27 28 29 30 31 32
		ACTION FUTURE COMPONENT SUBMITTED FORM SUB. SUPPLIER SUPP
	1 0	The cause was a component failure. The leaf spring anti-rotational device of the
	11	leadscrew nut assembly had fractured into several fragments, consequently, preventing
	1 2	the leadscrew from rising. The leadscrew nut assembly was replaced, and all fragments
	1 3	were retrieved. CR 5-8 was declared operable on 7/17/81 at 1020 hours.
1	1 4	All leaf springs were inspected during the 1984 outage and no problems were found.
	1 5	FACILITY STATUS SPOWER OTHER STATUS OTHER ST
		ACTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY 35 NA LOCATION OF RELEASE 36 NA N
	1 7	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 Ø Ø Ø 37 Z 38 NA
	7 8	9 11 12 13 80 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
	7 -8	9 9 9 40 NA 9 11 12 80
	1 9	LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION. (42) NA B501070513 B41217 Y STORY PDR ADOCK 05000346 PDR NRC USE ONLY
	2 0	ISSUED DESCRIPTION OF NA NA NA
D	VR 81	(110) 050 5000 5 . 050

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-81-45

DATE OF EVENT: June 25, 1981

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Inability to withdraw control rod 5-8 following reactor trip.

Conditions Prior to Occurrence: The unit was in Mode 3 with power (MWT) = 0 and Load (Gross MWE) = 0.

Description of Occurrence: During a routine recovery following a reactor trip, the Control Room operator discovered that control rod 5-8 was not moving while withdrawing group 5. Several attempts were made to latch and withdraw control rod 5-8 but the rod remained in the fully inserted position. Control rod 5-8 was declared inoperable at 0530 hours on June 25, 1981, which placed the unit in action item "a" of Technical Specification (TS) 3.1.3.1. Action item "a" requires the the shutdown margin requirement of Technical Specification 3.1.1.1 be satisfied, and that the unit be placed in hot standby within the next six hours. Both required actions were complied with.

Designation of Apparent Cause of Occurrence: The apparent cause of this occurrence was a component failure. The leaf spring anti-rotational device of the leadscrew nut assembly had fractured into several fragments, some of which lodged between the buffer spring assembly and the lead screw, which prevented the leadscrew from rising. No evidence was revealed to indicate the reason for the component failure.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. The unit was in the hot standby mode at the time of the event, and the existence of an adequate shutdown margin was verified. This fulfilled the action requirements of Technical Specification 3.1.3.1.

Based on prior operating experience with control rod drive mechanisms during which no leaf spring failures have been reported, Babcock and Wilcox feel that there is a low probability of a similar event occurring. Babcock and Wilcox performed a failure analysis on the leaf spring but results were inconclusive. All leaf springs were inspected during the 1984 refueling outage with no problems found.

Corrective Action: With the reactor plant cooled down, depressurized, and drained to support maintenance, a drive line check was performed on the C-7 control rod drive mechanism which normally operated control rod 5-8. The drive line check was conducted under Maintenance Work Order (MWO) 81-2676 to determine whether the binding problem was occurring in the core or in the drive mechanism. It was determined that the leadscrew was stuck since it could not be withdrawn while uncoupled from the control rod assembly. The C-7 control rod drive mechanism was removed under MWO 81-2717 and disassembled under MWO 81-2726. During disassembly,

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-45

it was discovered that the leaf spring anti-rotational device of the leadscrew nut assembly had fractured into several fragments, some of which lodged between the buffer spring assembly and the leadscrew, which prevented the leadscrew from rising. The leadscrew nut assembly was replaced during reassembly, and a new torque tube assembly minus the torque taker assembly, was installed due to damage sustained during disassembly when the snubber guide galled on the torque tube. Al! fragments from the broken leadscrew nut assembly were retrieved. The C-7 control rod drive mechanism was reassembled under MWO 81-2726 and reinstalled under MWO 81-2725. The drive line check was again performed on July 8, 1981 which demonstrated freedom of movement of the control rod assembly by raising the leadscrew and associated control rod assembly two inches with a hoist and spring scale. The control rod program verification was satisfactorily performed per Surveillance Test ST 5013.03 on July 15, 1981 and plant heatup commenced. Surveillance Test ST 5013.02, Control Rod Insertion Time Test, was performed and at 1020 hours on July 17, 1981, control rod 5-8 was declared operable, which removed the unit from Technical Specification 3.1.3.1, action item "a".

Failure Data: There have been no similar occurrences.



December 17, 1984

Log No. K84-1389
File: RR 2 (NP-33-81-45)
Rev. 1

Docket No. 50-346 License No. NPF-3

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Gentlemen:

LER No. 81-038 Rev. 1
Davis-Besse Nuclear Power Station Unit 1
Date of Occurrence: June 25, 1981

Enclosed is Licensee Event Report 81-038 Rev. 1 which is being submitted in accordance with 10CFR50.73, to provide 30 day written notification of the subject occurrence.

Yours truly,

Stephen m Franco

Stephen M. Quennoz Plant Manager Davis-Besse Nuclear Power Station

SMQ/bec

Enclosure

cc: Mr. James G. Keppler, Regional Administrator, USNRC Region III

> Mr. Walt Rogers DB-1 NRC Resident Inspector

> > IE22

JCS/001