

December 11, 1984

*DCE 016*

Docket Nos. 50-282  
and 50-306

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Mr. D. M. Musolf  
Nuclear Support Services Department  
Northern States Power Company  
414 Nicollet Mall  
Midland Square - 4th Floor  
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

We have completed the review of Revision 2 of your methodology reload report NSPNAD 8102P. The major revision to this report deals with a new accident analysis of the control rod drop event which we find acceptable. Thus the current restriction of the control rods to be manually operated at power levels above 90% when D bank control rods are less than 215 steps withdrawn is no longer required. This restriction can be removed from the Prairie Island Plant procedures after appropriate changes are made to the technical specifications. Other minor changes associated with Revision 2 of NSPNAD 8102P were also found acceptable.

We have also reviewed the error in COBRA IIIC/MIT Model setup as reported in your letter dated September 20, 1984. We agree with your conclusion that using the correction for the simplified channel input for wetted perimeter on card T2 does not affect the final transient parameter from the COBRA IIIC/MIT model setup to warrant concern since the corrected values are still well below the acceptance criteria.

A copy of our Safety Evaluation is enclosed.

Sincerely,

James R. Miller, Chief  
Operating Reactors Branch No. 3  
Division of Licensing

Enclosure:  
As stated

cc w/enclosure:  
See next page

ORB#3:DL  
PKreutzer  
*12/5/84*

ORB#3:DL *200*  
DDianni;ef  
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CPS *JR*  
LRubenstein/LKopp  
*12/10/84*

ORB#3:DL  
JRMiller  
*12/1/84*

Northern States Power Company

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