

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 3

TITLE (4)
10CFR50 Appendix "R" Cable Fire Protection

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)				
*			8 4	0 3 1	0 2	1 2	2 0	8 4						0 5 0 0 0				
														0 5 0 0 0				

OPERATING MODE (8)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
★		20.402(b)		20.406(e)		50.73(a)(2)(iv)		73.71(b)			
POWER LEVEL (10)	★	20.406(a)(1)(i)		50.38(e)(1)	Y	50.73(a)(2)(v)		73.71(c)			
		20.406(a)(1)(ii)		50.38(e)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text. NRC Form 365A)			
		20.406(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER	
R. L. Koenigs, Compliance Engineer	AREA CODE	
	5 0 9	3 7 7 - 2 5 0 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	B O C	B L	- - - -	N		B	B I	G B L	- - - -	N	
B	E B C	B L	- - - -	N		B	V A	G B L	- - - -	N	

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	05	31	85
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ABSTRACT (Limit to 400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

While performing a review and update of the WNP-2 Appendix "R" Safe Shutdown analysis, ten (10) cables required for safe Plant shutdown following a fire were identified on 4/12/84 as unprotected from fire (i.e., no thermolag fire protection material had been applied to these cables).

During a subsequent ongoing independent review of this analysis, on 11/6/84 another cable was identified as unprotected from fire over a short (15 ft.) length of vertical cable tray.

This same review identified four (4) additional cables as being unprotected on 12/5/84. Three of the four cables require the addition of thermolag coating to their associated conduits. The remaining cable requires wiring changes to include a transfer switch in the circuit to provide electrical isolation from the control room.

Engineering direction has been given for the application of thermolag coatings to original ten cables and the cable identified on 11/06/84. Plant Modification Records (PMRs) have been initiated to obtain Engineering direction for the deficiencies noted on 12/5/84. All cables were immediately placed on an hourly fire watch tour pending correction of the condition.

	<u>Date</u>	<u>Operating Mode</u>	<u>Power Level</u>
* Event 1	04/12/84	2	001
Event 2	11/06/84	1	095
Event 3	12/05/84	1	080

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PDR ADOCK 05000397
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039784	84	031	02	02	OF	03

TEXT (If more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

	<u>Event 1</u>	<u>Event 2</u>	<u>Event 3</u>
a) Power Level	1%	95%	80%
b) Plant Mode	2	1	1

Event

On 4/12/84, a review and update of the original 10CFR50 Appendix "R" Safe Shutdown Analysis identified a total of ten (10) cables which connected circuits necessary for safe plant shutdown that should have been protected (cable raceways with Thermolag fire protection coating) from an exposure to fire but were not. Nine (9) of the cables were added after the original analysis was complete. One cable was inadvertently missed by the original analysis.

Failure to protect these cables could have resulted in the loss of ability to safely shutdown the plant following a fire. These cables provided necessary indication and control for the Diesel Generator, Standby Service Water, Residual Heat Removal (RHR), and Reactor Building HVAC systems.

On 11/6/84, a second ongoing independent review of this analysis identified another cable that was not protected from fire over a 15 ft. length of vertical cable tray. Failure of this cable could have resulted in a loss of ability to isolate the reactor following a fire. This cable provided an isolation signal to a 1" RHR bypass valve (RHR-V-123A). All portions of this cable except the 15 ft. length mentioned above, were protected as a result of the original analysis.

On 12/5/84, the ongoing review identified four (4) additional cables which required protection. Three (3) cables require the addition of thermolag coating to their associated conduits. The fourth cable requires the inclusion of a transfer switch in the circuit to provide electrical isolation from the main control room under conditions where a fire exists in the main control room. These four (4) cables connected circuits necessary for safe Plant shutdown and should have been protected from exposure to fire, but were not. The cables provided necessary control and indication for the Diesel Generator, Standby Service Water, Residual Heat Removal and Reactor Building HVAC systems.

Immediate Corrective Action

Upon identification, each area was included on the Plant fire watch tour.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 9/31/85

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EXT (If more space is required, use additional NRC Form 385A's) (17)

Further Corrective Action

Engineering direction has been provided to thermolag all cables except those identified on 12/5/84. Field work is in progress implementing this direction and will continue as Plant conditions allow.

Plant Modification Records (PMRs) have been generated to obtain Engineering direction for correcting the conditions noted on 12/5/84.

Engineering will complete the review of the Appendix "R" Safe Shutdown Analysis by 4/30/85.

Safety Significance

WNP-2 experienced no fires in these areas, the Reactor Building Fire Detection System remained operable throughout the interval, and additional fire watch tours were performed on a routine basis. The health and safety of the public and Plant personnel was not endangered.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

December 20, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-031-02

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-031-02 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73, discusses the item of reportability, corrective action taken, and action taken to preclude recurrence, and provides supplemental information to LER 84-031-01.

Very truly yours,

C. M. Powers for
J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mmm

Enclosure:
Licensee Event Report No. 84-031-02

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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