From: David J. Vito (DJV)
To: JFH John Rogge
Date: Monday, March 27, 1995 10:03 am
Subject: RI-95-A-0026, QC Security Allegation

John,

I finally got a chance to look at the backup to the acknowledgement letter sent to the alleger by OE and try to get a chance to talk to him on Friday 3/24/95. I read through the documentation again to see if there appeared to be any DOL issues. I couldn't find anything that implied that he felt that his termination was due to his raising any safety issues to QC management or to the NRC, only that he was providing a "list" of all the problems he saw with the QC security program. You can go ahead and start processing the referral to the licensee now. Thanks.

CC: OGC, DFC, JFD

G/31

Talked to G. Smith on
3/27/95.
He does not see a [illegible]
of anything he deals with

G/32

ALLEGATION DISPOSITION RECORD Rev. 2 10/5/94

Site: Oyster Creek Section Chief (AOC): Rogge

Allegation No.: RI-95-A-0044 Date Received: 3/9/95

Acknowledged: n/a Receipt Report to SAC: 3/9/95

CONFIDENTIALITY GRANTED: Yes (No) OI Informed: yes

IS THERE A HARASSMENT/DISCRIMINATION ISSUE: Yes (No)
(If yes, complete H&ID section on reverse)
DOES THE ALLEGATION INVOLVE POTENTIAL WRONGDOING: Yes (No)
DOES THE ALLEGATION HAVE POLITICAL IMPLICATIONS: Yes (No)
DOES THE ALLEGATION REQUIRE RESOURCES TO RESOLVE WHICH CAN NOT BE OBTAINED BY THE AOC: Yes (No)

If yes to any of the above, the allegation needs to go to an Allegation Panel. Otherwise, document disposition actions below.

ALLEGATION PANEL (AP) DECISIONS

Date: 3/22/95 Previous APs on issue: Yes / (No)

Chair - Hanning via Branch Chief - —

Section Chief (AOC) - Briggs (actg) SAC - Vito

(Others) - G. Smith OI Rep. - ~~n/a~~ —

DISPOSITION ACTIONS: **(State specific action required for closure (including special concurrences), responsible person, ECD and expected closure documentation)**

1) Resident to inspect

Responsible Person: DRP/Rogge/Briggs ECD: 4/21/95

Closure Documentation: _____ Completed: _____

2) Closeout w/ memo to file

Responsible Person: SAC ECD: 4/30/95

Closure Documentation: _____ Completed: _____

3) _____

Responsible Person: _____ ECD: _____

Closure Documentation: _____ Completed: _____

4) _____

Responsible Person: _____ ECD: _____

Closure Documentation: _____ Completed: 6/33

Safety Significance Assessment: low

Allegation Receipt Report
(Use also for staff suspected wrongdoing)

Date/Time Received: * 3/6/95; 10:30 AM
3/15/95; 1:00 PM

Allegation No. ~~RI95A0058~~ RI-95-A-0058
(leave blank)

Employee Receiving Allegation or suspecting wrongdoing (first two initials and last name): G. M. Pindale

Name of Alleger: *

Home Address: *

Home Phone: *

City/State/Zip: *

Alleger's Employer: *

Alleger's Position/Title: * Maint Mechanic

Facility: Oyster Creek

Docket No. or Materials License No.:

Was alleger informed of NRC identity protection policy?	Yes (X) Yes PJA	No X
If a licensee employee or contractor, did they raise the issue to their management?	Yes X	No
Was confidentiality requested?	Yes	No X
Was confidentiality initially granted?	Yes	No X
Individual Granting Confidentiality:		

Criteria for determining whether the issue is an allegation:

Is it a declaration, statement, or assertion of impropriety or inadequacy? Yes / No

Is the impropriety or inadequacy associated with NRC regulated activities? Yes / No

Is the validity of the issue unknown? Yes / No

If No to any of the above questions, the issue is not an allegation and should be handled by other appropriate methods (e.g. as a request for information or an OSHA referral).

Allegation Summary or staff suspected wrongdoing (brief description of concern(s)):

Various issues were discussed related to work planning effectiveness, work union issues, delaying the individual from contacting the NRC, Oyster Creek management's dedication to doing permanant repair work vs. patchwork, and concerns relative to him being "singled-out" by OC supervision because he is outspoken regarding his concerns (including his willingness to talk to the NRC).

Number of Concerns: Several

Type of Regulated Activity
(a) X Reactor
(b) __ Vendor
(c) __ Materials
(d) __ Safeguards
(e) __ Other: _____ (Specify)

Functional Area(s):
___(a) Operations
___(b) Construction
___(c) Safeguards
___(d) Transportation
___(e) Emergency Preparedness
___(f) Onsite Health and Safety
___(g) Offsite Health and Safety
___(h) Other: _____

* These sections are Not completed for instances of potential wrongdoing identified by NRC staff.

Distribution: SAC OI

G/34

Detailed Description of Allegation or staff suspected wrongdoing:

On 3/8/95, the alleger telephoned the NRC Resident office and said that he had several concerns, however, his foreman would not allow him to talk to the NRC. I told him I would get back to him shortly. Immediately after our discussion, his foreman (Freeman), and other maintenance supervision telephoned the NRC office to inform me that earlier in the day had been very disruptive in a pre-job briefing. They said he expressed several outbursts over his concern regarding the use of station helpers in erecting scaffolding. They told me that was counseled for his disruptive action, and that he would probably be over to see me soon. It appeared that maintenance supervision delayed his visiting the NRC so that they can meet and contact the resident before the alleger arrived.

came to the Resident office around 10:30 a.m., and expressed several concerns related to work effectiveness and job pre-planning. He also stated that his foreman initially said that he could not approach the NRC with his concerns. The foreman (Bill Freeman) subsequently (about 10 minutes later) informed the alleger that it was O.K. to go to the NRC office, after he (Freeman) discussed actions with higher maintenance supervision.

On 3/7/95, the alleger was told to attend a pre-job briefing at 8:30 AM on 3/8/95. He was also told to arrive early to review the associated RWP. However, the alleger determined that the RWP and an associated job order were not completed. At the meeting, he observed that only mechanics were expected to perform the job. He thought that two station helpers would assist two mechanics, and therefore reduce radiation dose to mechanics. He then protested to the work assignment, and the lack of sufficient information to do an effective pre-job planning.

Two Radcon workers walked-in and asked "What's the problem." The pre-job meeting was then apparently postponed by Radcon due to lack of progress. However, Freeman told the maintenance workers to stay so they could continue to pre-plan. protested because there was no RWP or work order, and was planning from only a hand written sketch (job was to erect scaffolding).

went on to explain his frustration with Oyster Creek's mentality to do only patchwork to repair problems. He said workers are not allowed to do things right, and that many parts are no longer maintained in the storeroom due to major financial cuts. He characterized the above as a detriment to safe operation of the plant. I asked him if he had any examples of problems related to nuclear safety or those that could possibly adversely impact safe operation of the facility. He had none. However, he described one occasion when he and other workers were told to remove asbestos insulation although they were not qualified for asbestos removal. In that case, the work assignment was ultimately redirected, and qualified personnel performed the work.

also expressed concern that he was being singled out be his supervision because he is "an outspoken critic" of how business is conducted at Oyster Creek. He said that after the 3/8/95 incident, Freeman gave him a "brow-beating" concerning his "disruptive outbursts." asked Freeman for union representation prior to has "brow-beating," however, it was denied because Freeman said it was not disciplinary action.

also expressed concern regarding time he had taken off from work (about 2 weeks) as a result of some personal activities. He said he had requested time off without pay, however, they charged him with vacation pay. He requested correction, but it was not corrected. said although he showed a letter requiring his time away from the site to his acting foreman, supervision apparently denied seeing the letter and approving the time off. expressed concern that he was not being treated fairly, and that he is being single out because he is outspoken and talks to the NRC.

At the end of our interview, I explained to that I would process his concerns, and explained that some of his concerns fell outside NRC's purview, and that the NRC would get back to him.

One final note: On the afternoon of 3/8/95, the Maintenance Director (Jim Hildebrand) and the Radcon Manager (Roger Shaw) came into the NRC Resident office to 1) question whether I could discuss any results of my discussion with and 2) whether I had any further maintenance or radcon questions for them. I informed them that and I discussed the events of the morning (as they knew of), that I could not tell them any further information, and that I had no further questions at that point.

Update from R. Summers:

On the afternoon of 3/15/95, the alleger again telephoned the NRC Resident Office with the following additional information regarding apparent harassment and intimidation because of having raised the concerns about conducting the pre-job brief without a work package including a RWP, as follows:

1. He was taken to a disciplinary meeting regarding unapproved absences. This was for the aforementioned leave to appear in court. His acting supervisor had been given a letter from the individual's lawyer regarding the leave and individual claims that he received verbal okay. However, now individual was informed that management never approved or acknowledges receipt of individuals request for LWOP. Therefore, he was charged annual leave for the absent-period and penalized 2 days pay for discipline.
2. He has been assigned duties by himself while working nights, which is done to separate him from rest of workers.

The individual provided non-specific concerns regarding: drug use by O.C. managers and medical department personnel; abuse of craft urine samples (that are clean) to substitute for the samples from known managers who use drugs.

I then explained to the individual the NRC identity protection policy, especially in light of the individuals new claim regarding H & I. The individual explained that he had already brought this matter to management's attention and therefore he was not concerned that NRC might release his identity.

221 5
60617 1

EXHIBIT 1
DATA SHEET

4.4 Job Order No: 60617 1

Pressure Test Type: [ ] Hydro [ ] Pneumatic [X] Initial Service

System/Component Name: V-16-0103

Boundaries: WELD Connections n-o Seat Leakage

Spec. No.: _____ Spec Test Pressure: _____

6.1.1/6.2.1/6.3.1 Prerequisites met/Precautions Limitations understood: William K Green, 3-11-95
Job Supervisor Date

6.1.3.1/6.2.2.1 Pressure Gauge Serial No.: _____ Date Calibrated: _____

Observed Pressure Gauge Reading: _____

Pressure Gauge Calibration Correction: NA

Actual Corrected Pressure: _____

6.1.5.1/6.2.5.1/6.3.3.1 Leakage observed: [ ] Yes [X] No

Leakage description: _____

Testor: William K Green, 3-13-95
Signature Date

QA/QC: _____
Signature Date

ANI: NA
Signature Date

6.1.7.1/6.2.6.1 Test set-up removed: _____
Job Supervisor Date

_____
Verified By Date

6.1.8/6.2.8 Tests Results: SAT

7.1 Pressure Gauge check [ ] Sat [ ] Unsat

Comments: NA G/35

GPU NUCLEAR

TRANSMITTAL SHEET

WELDING ENGINEERING PACKAGE IMPLEMENTATION

SECTION I:

TRANSMITTAL NO. 1 SHEET 1 OF 1

JO# 60617

INSTALLATION SPEC.# WPIR / PE 125-1 FILE# 072-95 REV. 0/0

BUDGET ACTIVITY# 312400 CLEAN-UP VALVE V-16-103

SECTION II: APPLICABLE WELDING DOCUMENTS

WELD/BOND MAP NO./REV.	WELD/BOND NUMBERS
3B-215-WM-012 / 0	ISSUED WELDS# 215-051 THRU 215-056
TOTAL PIPE WELDS/BONDS ISSUED	6

STRUCTURAL MAP NO.	APPLICABLE WELDS
N/A	N/A
TOTAL STRUCTURAL WELDS/BONDS ISSUED	N/A

SECTION III: WELDING/BONDING ENGINEERING CHECKLIST

1 ■ WELD MAP ☐ REV. WELD MAP ☐ WELD HISTORY RECORD

☐ WELD REPAIR RECORD 2 ■ PIPE WELD RECORD ☐ STRUCT. WELD RECORD

☐ AI/ANI/ANII ☐ BOND MAP ☐ PIPE BOND RECORD

SECTION IV: REMARKS:

SECTION V: ALL APPLICABLE WELDING DOC'S HAVE BEEN REVIEWED

WILLIAM R. CRIQUI 03/08/95 DM 3-9-95

QC WELDING/BONDING ENGINEERING SIGN/DATE

FORM 125-1

ENGINEERING EVALUATIONS

Prepared By: C.A. Schilling

Date Prepared: MARCH 2,1995

Check As Applicable:

* Alternate Replacement per 124.2 X
* Configuration or Facility Change per EMP-002
* Engineering Direction/Evaluation

PSETA/Group No. NA

System ID No.(s) 215

B/A No. NA

Work Request No. 766616

Job Order No. 60617

Deviation RPT No. 95-109

Description: V-16-0103 REPLACEMENT

The system/component related to this activity is classified as:

NSR____ RR____ OTHER X

Lonergan relief valve model FSS-1 is no longer made. An acceptable replacement is Lonergan series L14B 1" x 1" st.st relief valve. The old valve had socket weld connections vs. threaded connections for the new valve. B31.1 does not allow the use of threaded fittings for water service above 100psi and 220°F. Although the design conditions for line specifications DR-13 and DR-15 are 150psi and 250°F the threaded connections are permitted for the following reasons:

1. Normal operating conditions are 110psig and 120°F or less.
2. The system is instrumented to not exceed the operating conditions.
3. This is a 'thermal' relief valve. Its purpose is to protect the Demineralizer in the event that the Demineralizer is full and isolated and the water expands.
4. The inlet threaded connection will be seal welded.

A threaded coupling may be used to adapt the threaded inlet of the valve to socket weld. One end of the coupling is to be bored to a diameter of 1.330"+/-.005" and a depth of .5" to .65". The coupling is to be treaded onto the inlet of the valve and the threads seal welded. CAUTION: Minimize the heat buildup in the valve during seal welding.

Prepared by: C A Schilling Date: 3-6-95
Signature

Independent Review Required: X YES ____ NO Init. CAS
(Required for NSR Evaluations and NSR/RR Alternate Replacements)

Independent Reviewer: [signature] Date: 3/6/95
Signature
(Must be RTR for Alternate Replacements)

Reviewed and Accepted By: [signature] 3/6/95

125-1 CONTINUATION SHEET

The outlet of the valve does not have to be seal welded because it goes to a hub drain.

A Fire Hazard Input and Status Form is not required because no combustibles are being added and there are no changes or interfaces with any fire protection feature or system.

An EQ Input and Status Form is not required because there are no EQ requirements for this component.

When the installation is complete submit an FCN to revise the GMS2 component data base with the new valve nameplate data.

Eng. Evaluation # 072-95

ATTACHMENT 124.2-1

ALTERNATE REPLACEMENT EVALUATION

1) Does this activity meet the definition of an "Alternate Replacement" X YES ___ NO

Describe the "Alternate Replacement" and why it is required. Relief Valve V-16-0103 is obsolete and needs to be replaced. Therefore a new valve must be specified.

2) Safety Determination/Evaluation attached. CAL Initial

3) Detailed evaluation of alternate replacement attached. (125-1 Form) CAL Initial

4) Technical Evaluation of Replacement Item. (124.2-2, 124.2-3) CAL Initial/ NA

5) Do Engineering Configuration Concerns of EMP-014 apply?	YES	NO	Concurrence Obtained (If Yes)
Masonry Block Walls	___	✓	
Electrical Loads	___	✓	
Piping and Pipe Supports	___	✓	
Raceway Loads	___	✓	
Environmental Qualification of Electrical Components	___	✓	
Fire Hazards Analysis Report	___	✓	
Containment Electrical Penetrations	___	✓	
Control of Concrete Rebar Cutting	___	✓	
Plant Computer Configuration	___	✓	
Human Factors	___	✓	
RF Interference and Electrical Noise	___	✓	
Appendix R Commitment	___	✓	
Tech Functions concurrences obtained for each item marked "YES"	___	✓	NA Init / NA

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ATTACHMENT 124.2-1

ALTERNATE REPLACEMENT EVALUATION
(Continued)

6) FCN required ✓ YES ___ NO

~~FCN # _____ issued or~~ implementor directed by 125-1 Form to issue upon installation. CAS Init/NA

7) Procedure Changes Required (attach list if required). NA Init/NA

8) Preventive Maintenance Update Required (attach PM Requests). NA Init/NA

9) Vendor Manuals Update Required (attach request). NA Init/NA

10) Spare Parts Required (attach list). NA Init/NA

11) NPRDS Data Base Update NA Init/NA

12) Form 124.2-2 issued. Copy attached ✓ Issue later ___ CAS Initial

13) Cross Disciplinary Review Required. NA Init/NA

14) References WR/JO # 60617 PETA/Item No. NA

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ATTACHMENT 124.2-2

REPLACEMENT ITEM TECHNICAL EVALUATION FORM

1. Item Identification

Tag No.: V-16-103

	Manufacturer	Model/Part #	Description
Original	Lonergan	F55-1	1" St. St. Relief Valve
Replacement	Lonergan	L148-F8-M60150	1" SS. Relief Valve

2. Function/Location
'Thermal' Relief Valve on the Cleanup Demineralizer inlet.

3. Functional Classification: Other (NSR, RR or "Other")

Basis: ✓ QCL ___ QA Screen/DKPS ___ Other (Attach documentation)

4. Functional Mode: _____ (Active or Passive)
Basis: _____

5. Credible Failure Mode(s)/Failure Effects (NSR - only)
Set point drift

6. Critical Characteristics for Design/Acceptance
materials of construction, inlet/outlet size, inlet to outlet diam.

7. Equivalency Evaluation (See attached Equivalency Evaluation Table)
See Attached

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ATTACHMENT 124.2-3

EQUIVALENCY EVALUATION TABLE

Comparison Codes:

I = Identical to original design requirements
E = Exceeds original design requirements
R = Design difference reconciled in Remarks section
(For NSR or RR Replacements with NSR interfaces, ensure differences are reconciled with design basis documents)

CHARACTERISTIC	ORIGINAL	REPLACEMENT	COMPARISON
~~Body CRD~~	~~EE 904 Rev~~		
INLET	1" SW	1" NPT	I *
Outlet	1" SW	1" FPT	I *
Setpoint	150 PSI	150 PSI	I
Inlet to & outlet	2½"	2½"	I
Spring	SS. SS.	SS. SS.	R1
Materials of Construct.	SS. SS.	SS. SS	I
			I

Remarks: R1 Field Fit-up to compensate

* SEE 125.1 FOR RECONCILIATION OF THREADED VS SOCKET weld.

Design Basis Reconciliation (NSR/NSR Interface Replacements)

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ATTACHMENT 124.2-4

CONFIGURATION CHANGE NOTIFICATION

Description:

Status: ☐ Installed WR/JO # _____

☒ Evaluation Complete to be installed by WR/JO # 60617

☐ Evaluation Complete outage required to install WR/JO # _____

Turnover: Operations requested to perform Procedure 124 Turnover

☐ Yes ☒ No

Operations Contact: B. Shumaker

	Yes	Complete	No
Procedure changes required			✓
Procedure cross-disciplinary review required			✓
Preventive Maintenance update required			✓
Vendor Manuals update required	✓		✓
FCN required	✓		
Spare Parts required			✓
Labels per Procedure 109 required *			✓
NPRDS Data Base update			✓

CDS 3-6-95

* (Note: If yes and not complete, attach list and status)

Distribution

Manager Operations Support
Vendor Document Control Manager
Director Engineering & Design
Supervisor Procurement Engineering
System Engineer
Manager Maintenance Assessment
Manager Plant Training Oyster Creek
Quality Control Manager

From:

Originator: C. Schilling ext. 4907
Supervisor: [illegible] 4895
Manager: [illegible] 4081

Eng. Evaluation 072-95 Rev. 0

ATTACHMENT 124.2-5

MATERIAL REQUIREMENTS

ITEM DESCRIPTION	QTY REQ'D	NSR Y-N	PQC	SSN/P.R. NUMBER
Lonergan 1"x1" St. St. relief valve	1	N	N/A	95-7240-95-0397-000

Procurement Engineer Review: Signature _____ Date 3/1/95

(3095P/14)

Safety/Environmental Determination and 50.59 Review
(EP-016)

Unit: Oyster Creek

Document/Activity Title: V-16-0103 Replacement

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Document No. (if applicable): 072-95 Doc. Rev. No. 0

SE Rev. No.

SE No.

Type of Activity (modification, procedure, test, experiment, or document): Alternate Replacement

1. Does this document involve any potential non-nuclear environmental concern? ☐ Yes ☒ No

   To answer this question, complete the Environmental Determination (ED) form. Any YES answer on the ED form requires an Environmental Impact Assessment by Environmental Controls, per 1000-ADM-4500.03. If in doubt, consult Environmental Controls or Environmental Licensing for assistance. If all answers are NO, further environmental review is not required. In any event, continue with Question 2, below.

2. Is this activity/document listed Section I or II of the matrices in Corporate Procedure 1000-ADM-1291.01? ☒ Yes ☐ No

   If the answer to question 1 is NO, stop here. This procedure is not applicable and no documentation is required. (If this activity/document is listed in Section IV of 1000-ADM-1291 review on a case-by-case basis to determine applicability.) If the answer is YES, proceed to question 3.

3. Is this a new activity/document or a substantive revision to an activity/document? (See Exhibit 2, paragraph 3, this procedure for examples of non-substantive changes.) ☒ Yes ☐ No

   If the answer to question 3 is NO, stop here and complete the approval section below. This procedure is not applicable and no documentation is required. If the answer is YES, proceed to answer all remaining questions. These answers become the Safety/Environmental Determination and 50.59 Review.

4. Does this activity/document have the potential to adversely affect nuclear safety or safe plant operations? ☐ Yes ☒ No

5. Does this activity/document require revision of the system/component description in the FSAR or otherwise require revision of the Technical Sepcifications or any other part of the SAR? ☐ Yes ☒ No

6. Does the activity/document require revision of any procedural or operating description in the FSAR or otherwise require revision of the Technical Specifications or any other part of the SAR? ☐ Yes ☒ No

7. Are tests or experiments conducted which are not described in the FSAR, the Technical Specifications or any part of the SAR? ☐ Yes ☒ No

   IF ANY OF THE ANSWERS TO QUESTIONS 4, 5, 6, OR 7 ARE YES, PREPARE A WRITTEN SAFETY EVALUATION FORM.

   If the answers to 4, 5, 6, and 7 are NO, this precludes the occurrence of an Unreviewed Safety Question or Technical Specifications change. Provide a written statement in the space provided below (use back of sheet if necessary) to support the determination, and list the documents you checked.

   NO, because: See Attached

   Documents checked: FSAR 5.4.8; TS. 3.3.E

8. Are the design criteria as outlined in TMI-1 SDD-T1-000 Div. I or OC-SDD-000 Div. I Plant Level Criteria affected by, or do they affect the activity/document? ☐ Yes ☒ No

   If YES, indicate how resolved:

APPROVALS (print name and sign)

Engineer/Originator	S.A. Lahilling	
tion Manager		
nsible Technical Reviewer		Date 3-3-95
Other Reviewer(s)		Date
		Date

N5047 (04-93)

GPU Nuclear

ENVIRONMENTAL DETERMINATION

Division/Unit	Document No.	Revision No.
Oyster Creek	Eng. Evaluation 072-95	0

Document Title

V-16-103 Replacement

1.	Will Document implementation result in an increased potential to release hazardous chemicals (gas, liquid, solid or semi-solid) to the environment?	☐ Yes	☒ No
2.	Will Document implementation compromise existing capability to control, treat, or monitor releases to the environment?	☐ Yes	☒ No
3.	Will Document implementation cause a physical or chemical change in the characteristics of facility discharges, effluents, or withdrawals?	☐ Yes	☒ No
4.	Will Document implementation result in the permanent or temporary storage (for use, disposal, or transfer) of any hazardous or other regulated waste, or hazardous chemical(s), outside of established handling facilities or procedures where the margin of control or containment will increase the potential of a release to the environement?	☐ Yes	☒ No
5.	Will the implementation of the Document result in an increase in the amount or a change in the type of hazardous waste(s) typically generated, and/or previously evaluated for the type of activity?	☐ Yes	☒ No
6.	Does Document implementation result in land disturbance (e.g., excavation work, grading), or modification or alteration of storm water drainage systems that would change site storm water runoff or increase sediment loading of storm water runoff?	☐ Yes	☒ No
7.	Will Document implementation result in a physical alteration to a wastewater treatment facility or other facility system(s) or component regulated by environmental permits (e.g., discharge to groundwater permit, discharge to surface water permit, etc.)?	☐ Yes	☒ No

SIGNATURES

Preparer: CA Sabilling	Date: 3/3/95
Cognizant Supervisor/Manager, or Designee	Date
Environmental Technical Reviewer	Date
Other Reviewer(s)	Date

Determination/Evaluation Continuation

4. This activity will restore the Cleanup System integrity. A leaking relief valve is being replaced with a relief valve of the same setpoint and size.

5. This valve is not described in detail in any licensing basis document. A 1" x 1" stainless steel relief valve is being replaced by a 1" x 1" stainless steel relief valve with the same setpoint.

6. This is a passive valve. It does not require operator action at any time.

7. This is not a test or experiment.

PRE-JOB BRIEFING CHECK LIST

RWP 950096 DATE 3/9/95 TIME 0830

RCFO: S. WHALEY (print) initials

	Initials
RWP / ALARA REVIEW REQUIREMENTS: Confirm that the involved Radiological Controls Technicians (RCTs)s have read or reviewed the RWP and ALARA review (if any).	
RADIOLOGICAL CONDITIONS: Review the Dose-Rates, Contamination Levels and Airborne Radioactivity levels in each of the work areas identified above. Maximum Dose Rates, Low Dose Waiting Areas, Dose stratification (Non-uniform dose rates), and other identified Radiological Parameter are to be reviewed.	
CHANGES TO RADIOLOGICAL CONDITIONS: Explain the anticipated changes (if any) to the Radiological conditions in each of the work areas caused by this task or other tasks working in the area.	
ENGINEERING CONTROLS: Review the Engineering Controls, in-place or planned, (HEPA Exhaust system(s), HEPA Vacuum cleaning, containments, catches, drains, shielding, etc.).	
RWP DOSIMETRY REQUIREMENTS: Identify any SPECIAL dosimetry requirements specified on the RWP and confirm that all personnel understand their individual dosimetry requirements.	
REQUIRED AVAILABLE EXPOSURE: Identify the required available exposure from the RWP. The GRCS/RCT will confirm that all personnel understand the required available exposure limit and that they are aware of the approximate dose they expect to receive.	
RWP PROTECTIVE CLOTHING REQUIREMENTS: Identify any SPECIAL protective clothing requirements specified on the RWP and confirm that all personnel understand their specific ProtectiveClothing Requirements.	
COVERAGE / MONITORING: Review the monitoring requirments specified on the RWP and identify surveys and other Radiological Controls actions specified in the Coverage Plan. Confirm that the workers and the Covering RCTs understand the Monitoring requirements and specific survey actions.	
RWP RESPIRATORY PROTECTION REQUIREMENTS: Confirm that all personnel understand their specific Respiratory Protection Requirements and Breathing Zone Air Sampling requirments (if any). Confirm that workers understand that MPC (DAC) hours may be acrued during this task and that they are aware of the approximate number expected.	

Remind crew/workers to use "self check"

RCFO All questions have been resolved with the following noted exceptions: SELF CHECK DISCUSSED G/36

[ ]-NONE (Sign)

ALARA/FORMS1/7

RWP 950096 DATE 3/8/95 TIME: 0845

Job Supervisor: Steve Consi
(or designee) (print) initials

Initials

RWP / ALARA REVIEW REQUIREMENTS: Confirm that all workers have read/reviewed the RWP and ALARA review (if any).

SCOPE: Describe the scope of work to be performed which is to be covered by this briefing. NOTE: The entire scope of work need not be briefed if a series of incremental briefings (shiftly, daily, etc.) are to be done.

WORK AREAS: Identify all the work areas inside the RCA. All laydown areas or temporary work areas are to be identified. The location(s) where material is to exit the RCA is to be identified (MAC, Bldg-3, Gate 20, etc.).

INTERFERENCE / COORDINATION: Identify any other tasks known to be working in the same area(s) and the possible impact that these other jobs may have on this task.

SYSTEM STATUS: Identify the status of the systems involved in the task(s). (tagged-out and drained, in-service, not-drained, energized, pressurized, etc.).

SYSTEM OPENING / OPEN SYSTEM WORK: Identify all system opening evolutions or locations (components) where open system work is to be done. If any draining of fluid systems is to be done, identify where and how the drainage is to be routed.

WORK PROCESS: Identify the work processes to be done (welding, burning, grinding, hand-tools, power tools, test equipment, lapping, hydrolazing, etc.).

WORKER POSITIONS: Individual workers will identify their expected work positions (standing, kneeling, etc), and all locations where the workers expect to go.

SAFETY: Confirm that there are no unanswered Industrial Safety Concerns (heat stress, fall protection, asbestos, temporary use of power cords, confined space etc.

Job Supervisor: All questions have been resolved with the following noted exceptions: G/37

_____ [ ]-NONE (Sign)

GPUN

WELDING PACKAGE INFORMATION REQUEST

JOB ORDER#: 6417 BA#: 316400

PLANNER: T Clarke EXT: 1318 DATE: 3/8/95

DATE REQUIRED: 3/9/95 DATE RECEIVED BY WELDING: 3/9/95 mc

DESCRIPTION OF WORK: Install new Relief Valve V-16 0103

DATE RETURNED: 3/6/95 mc 1. _____ 2. _____

DATE RETURNED FOR CORRECTIONS: 1. _____ 2. _____

REASON FOR RETURN: 1. _____

2. _____

DATE RETURNED TO WELDING: 1. _____ 2. _____

MATERIAL INFORMATION

PIPING

ITEM NO.	SIZE	SCH.	SPEC/GRADE	NEW/EXIST
1.	1" SS	40	A312 316 SS	X
2.	1" CS	40	A106 GR.B	X
3.	1" CS	40	A106 GRB	X
4.	1" SS	40	A312/A376 TP304/TP316	X
5.				
6.				
7.				
8.				

PIPE FITTINGS/VALVES

	TYPE	SCH/RATE	SPEC/GRADE	NEW/EXIST
9.	ELBOW 45° CS	3000#	A105	X
10.	Coupling 1" SS	3000#	A182 F316	X
11.	Coupling 1" THDD	3000#	A182 F316	X
12.				
13.	Valve Relief 1x1	150#	A351-CF8M	X
14.				
15.				
16.				

STRUCTURAL/OTHER

	TYPE	SCH/RATE	SPEC/GRADE	NEW/EXIST
17.				
18.				
19.				
20.				
21.				
22.				
23.				
24.				

6138

JOB: 66617

ENGINEERING INFORMATION

SYSTEM NAME: Cleanup System

SYSTEM NO.: 215 LINE SPEC: ~~BB-13~~ NB & ~~BB-15~~ NB DM PTCM WRC 3-9-95

SAFETY CLASS: NSR ☐ RR ☐ OTHER ☒

ASME SECTION XI: YES ☐ NO ☒

CLASS: IWB ☐ IWC ☐ IWD ☐ N/A ☒

REPAIR PROGRAM: YES ☐ NO ☒

CONSTRUCTION CODE: B31.1 YEAR: 1955

INSPECTION CRITERIA

INSPECTION CODE: B31.1 YEAR: 1989

NDE REQUIRED: VT ☒ MT ☐ PT ☐ RT ☐ UT ☐

LEAK TEST ☒ HYDRO TEST ☐ OTHER ☐ _____
Non-Isolatable from relief valve

ANI/ANII REQUIRED: YES ☐ NO ☒

STRUCTURAL MATERIAL TRACEABILITY: CMTR ☐ C OF C ☐ N/A ☒

PIPING MATERIAL TRACEABILITY: Spec & grade

OTHER MATERIAL TRACEABILITY: _____

ATTACHMENTS: MNCR ☐ PCR ☐ FCN ☐ OTHER ☐ _____

_____ ENGINEER/DATE

A.L. Schilling 3/3/95 ENGINEERING APPROVAL/DATE

NOTES: Ref Plant Engineering 125.1 # 072-95

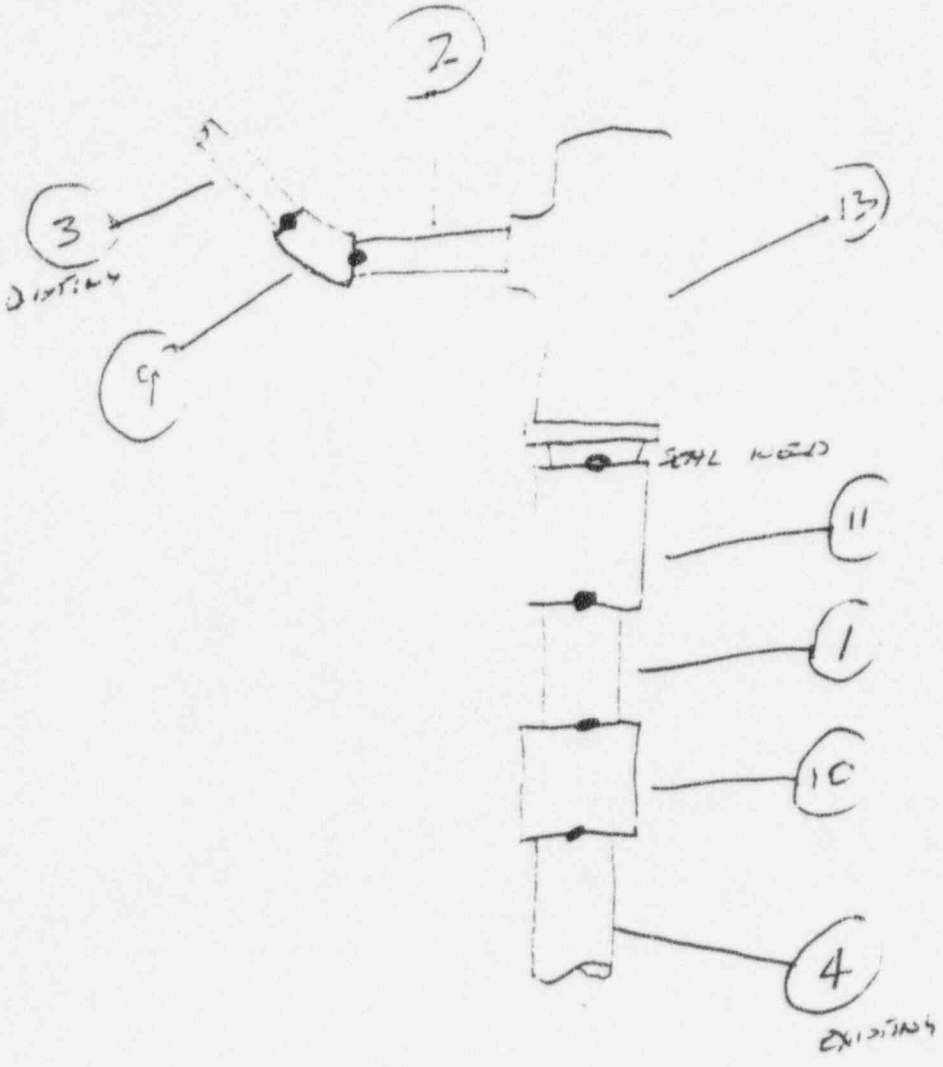
PRE-APPROVED ENGINEERING USED FOR THIS WPIR ☐

_____ PLANNER/DATE

WPIR SKETCH

JO#: 6617

WELD MAP#: 3B-215-MM-C12



FORM 6630-ADM-4010.02-1

RADIOLOGICAL REVIEW REQUEST FORM

I

Title Activity Description	Document No.	RER
RxB 75' CLEAN-UP VALVE AISLE REPLACE V-16-103	JA# 60617 312400	95010B

REVIEW REQUEST DATA		ARN REVIEW REVISIONS			EXPOSURE MANAGEMENT
Requestor's Name	R. HEFFNER/CLAFFEY	Rev.	Date	Rad Con Signature	ETN
Homebase Code	2110				RFD16VM
Telephone Number	4496 / 4712				ESTIMATED EXPOSURE (person-Rem) 1.205 1.0
Review Need Date	3-8-95	REVIEW DATA USED			ACTUAL EXPOSURE (person-Rem)
RAD CON RECEIVING DATA					EXPIRATION DATE
Date Time	3-8-95				12-31-95
Review Basis	DOSE ESTIMATE				
ACCEPTED BY:	Robert A Heffner	Maximum Exposed Individual (Per RWP Entry) 400 mRem			

DOSE ESTIMATE ONLY

II

RWP TIME STUDY			DESCRIPTION OF RWP AREAS			
			1	2	3	4
	NC	Job Classification	75' VALVE AISLE	75' OUTSIDE		
A		MECH SCAFFOLDING INSTALL&REM	16 8	8		
B		STA.SER/MECH SHIELDING	1.0			
C		STA.SER/MECH CLEAN-UP	.25			
D		MECH WELDING	2.5			
E		FIRE WATCH	2.5			
F		RAD CON	2			
G						
H						
Max. Time Per Individual Entry			16.25	8		

Review Performed By

Robert A Heffner 3-8-95
(Rad-Eng) ROBERT A HEFFNER Date

Concurred By (if applicable)

J. DERBY / TRD 3/8/95
Date

6139

OFFICIAL

GPU NUCLEAR
JOB ORDER

JO# 00060617 CYCLE 15 WORK TYPE CM
PRI 1 MLSTN NA REVISION 00
WR# 766616 STATUS AUTH REV DATE 03/07/95
TIME SHT# 2400-CRB12

WORK REQUESTING INFO

Loc Desc
Comp V-16-0103 VALVE REACTOR BLDG 75 FT 3 IN L
Comp Desc RWCU SYSTEM DEMINERALIZER INLET SAFETY RELIEF VALVE SYS 215

Work Desc THE RELIEF VALVE IS LEAKING-BY. PLEASE INVESTIGATE AND REPAIR.
(DVR 95-109)
Cause of Failure & Symptoms

REFERENCE USE
AW PROC. 2400 - ADM - 1213.00

ETN=RFD16VM/PLANNER PLEASE CONTACT RAD-ENG R.HEFFNER X-4[illegible] HANKS JDERB
Y-950228

Failure Codes
Detect Symptom

WORK INITIATION APPROVALS

Title	Signature		Date	Route#
Originator	JEFF DOSTAL	Ext 4656	02/27/95	MP06
OPERATIONS MANAGEMENT	COLANGELO, M.		02/28/95	
WORK AUTHORIZATION APRV	COLANGELO, M.		02/28/95	
MAINTENANCE JOB PLANNER	CLIFFEY J.		03/07/95	
AREA SUPERINTENDENT			3-9-95/10'9Am	
J/G COORDINATOR			3/9/95	
GSS TO START WORK			3/4/95	

* For prep activities only

Attachment #
JO #
Page of

Ops Comments:

WORK COMPLETE APPROVALS

Title	Signature	Date/Time
J/G COORDINATOR		3-14-95 1012
QA AFTER JOB COMPLETE	NA	
GSS AFTER JOB COMPLETE		3-14-95 1021
AREA SUPERINTENDENT		4/10/95 0845
MAINTENANCE ASSESSMENT		
DOCUMENT CONTROL		

6/40

OFFICIAL

GPU NUCLEAR
JOB ORDER

JO# 00060617 CYCLE 15
PRI 1 MLSTN NA
WR# 766616 STATUS AUTH

WORK TYPE CM
REVISION 00
REV DATE 03/07/95
TIME SHT# 2400-CR812

PLANNING INFO

Loc Desc
VALVE REACTOR BLDG 75 FT 3 IN L

Comp V-16-0103
Comp Desc RWCU SYSTEM DEMINERALIZER INLET SAFETY RELIEF VALVE SYS 215

Work Desc THE RELIEF VALVE IS LEAKING-BY. PLEASE INVESTIGATE
AND REPAIR.
(DVR 95-109)

NSR N RR N OTHER Y OQA PLAN Y ISI N CLEAN CLASS B NPRD COMP N
PLANT MODE _____0 ENV QUAL N IST N SAFETY CLASS

Special Comments
REACTIVITY ITEM

Weld/Bond Docs Reqd?	W	Switch/Tag/Drain Reqd?	Y	Work Cntr	ASME
Grind/Burn Permits Reqd?	Y	Temp Variation Reqd?	N	Craft	MECH
Auth Nuclear Inspec Reqd?	N	Deficiency Tag Issued?	P	Est Mnhrs	0032
Insurance Insp Prgrm Reqd?	N	Fire Barrier Breached?	N	Crew Size	04
Resp Technical Rev Reqd?	A	Ind Safety Rev Reqd?	N	EM#	072-95
ALARA Review Reqd?	Y	Confined Space Reqd?	N	EER#	
Radiation Wrk Permit Reqd?	Y	Non-Rad Protect Reqd?	N	DOC#	
Leak Ctmt Installed?	N	Scaffolding Reqd?	Y	QDR#	
Insulation Removal Reqd?	N	Post Maint Test Reqd?	Y	MNCR#	
ARN 95010B	ETN RFD16VM	Plt Engineer		Prj Eng	
Est Milirem	1600	BA 312400	WO 2400-50187	WA	240050187
QV Required N					
QV H/W Issued		Notif Date / /			
QV Notif Name					

---------- Procedures ---------- --Tech Manual-- ---- Drawings---

COMPONENT BASE LINE INFORMATION

		Comp Size	SCEW Sht#	Matls IDN
TF Verify	Y			
Comp Code	VALVE	1.0X1.0	N/A	000000000
Comp Type	PRESSURE RELIEF	Reference	MFG-LG B-2402	
PWR Source	N/A	Documents &	FLO-GE 148F444 SH1	
Pri Src Doc	MPR2400-SMM-3218.04	Drawings	ECI-N/A	
QA/QC Rqmt	QA/QC REQUIRED	Seismic Func	NO SEISMIC FUNCTION	
QA/QC Bas	COMP RELIABILITY	Func Mode	AUTOMATIC OPERATION	
Func Class	PLANT SUPPORT	Classific	OTHER	
Mfr Name	LONERGAN, JE COMPANY	Model#	FSS-1	
Mfr Code	L265	Serial#	N/A	

OFFICIAL

JO# 00060617 JOB ORDER WORK PERFORMED TIME SHT# 2400-CR812

Problem Condition Found Relief VALVE PASSING

Corrective Action/Special Tools, Rigging etc.

STAGED AND BUILT SCAFFOLD

MADE FIELD MEASUREMENTS

CUT AND BENCH WELDED VALVE AND ASSOCIATED PIPING

CUT OUT OLD VALVE

INSTALLED NEW VALVE ASSEMBLY AND MADE 2 FIELD WELDS.

QC INSPECTED AND ACCEPTED WELDS

PULLED TAGS 0410

SUBMITTED FCN

① Scaffolding and Portable Shielding will be Removed under ASME-MM-004703 to allow for time Review and closeout of the welding Documents.

Post Maintenance Test SAT

Responsible Supv Work Center ID? ASME Run Hour Meter Reading? NA

Environmental Qual Restored? YES (N/A) Shift Completed? DAY (MID) NIGHT

NPRD Failure Report Required? (YES) N/A Results Code OT

Component Failed? (YES) NO Leak Containment Removed? YES (N/A)

Employee ID# 02861 09009 C227 9470 B351 0016

or

Contract Badge# 4609 A492 B976 ____ ____ ____

OFFICIAL

JO# 00060617 JOB ORDER MACHINERY HISTORY TIME SHT# 2400-CR812

Cause of Failure (Root Cause) Description DUE

CAUSE OF FAILURE DESCRIPTION CODES

Lubrication Problem- AE	Aging/Cyclic Fatigue - (BD)	Insulation Breakdown- AR
Weld Related - AF	Dirty - BE	Setpoint Drift - AL
Mech Damage/Binding- BB	Previous Repr/Install- AM	Out of Calibration - BH
Mech Adjustment Out- BC	Incorrect Procedure - AN	Short/Grounded - AS
Normal/Abnorml Wear- AD	Incorrect Action - BJ	Open Circuit - AT
Abnormal Stress - AG	Loose Part/Connection- AV	Contacts Burn/Pitted- AU
Foreign Material - AB	Material/Part Defect - AZ	Circuit Defective - AW
Blocked/Obstruction- BF	Wrong Part - AA	Burned/Burned Out - AX
Corrosion - BG	Other - (XX)	Electrical Overload - AY
Particulate Contam - AC		

** CHOOSE ONLY THE **
BEST THREE CODES

CORRECTIVE ACTION CODES

Recalibrate/Adjust - AA	** CHOOSE ONLY **	Repair Component/Part-AG
Temporary Measures - AC	ONE CODE	Replace Component -(AK)
Modify/Substitute - AE		Replace Parts -AH

ENGINEERING SUPPORT INFO

** Please Record if not Available with Job Order Issuance **

Comp Data Base Update Reqd? (YES) N/A

New Model# _____ New Serial# _____

New Mfr _____

Eng Mod# 072-95 Eng Eval Report# 125 0712-95 Memo# _____

~~MNCR#'s~~ _____ _____ _____ QDR#'s _____ _____ _____

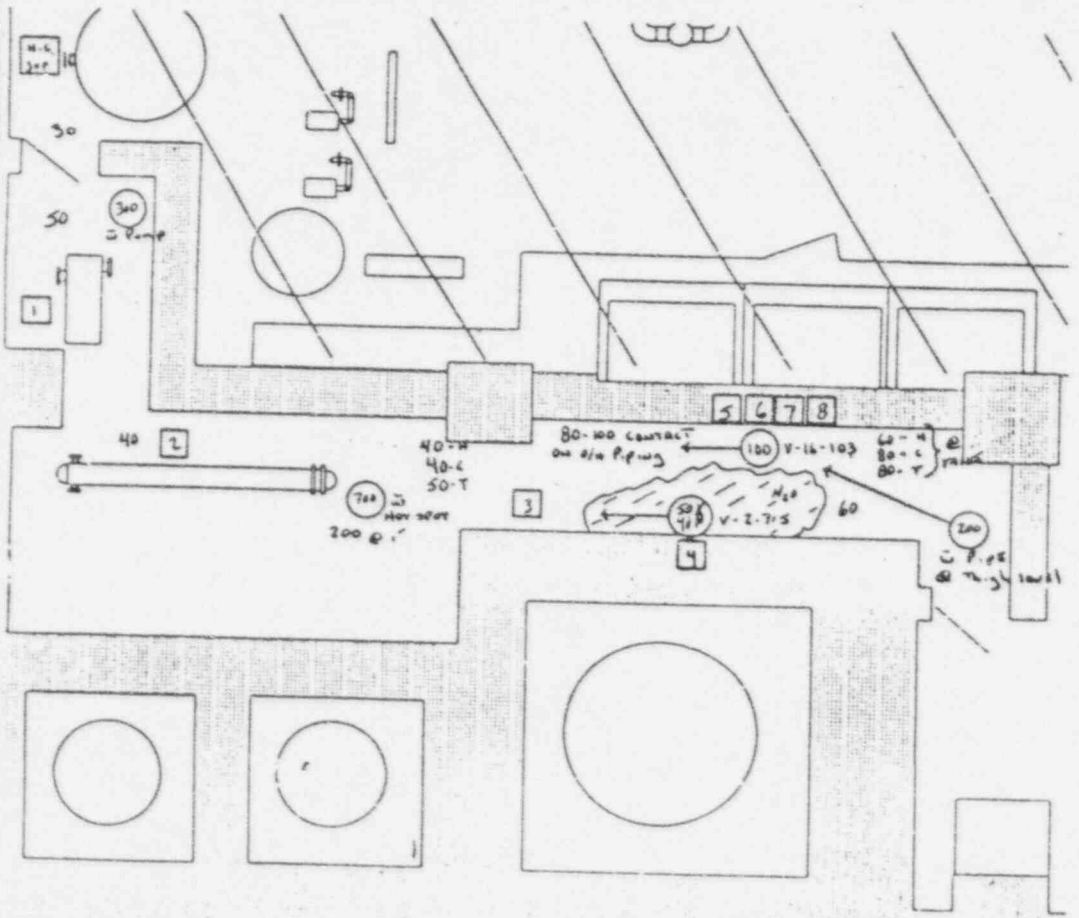
RWP# 950096 STD# 95-029

GPU Nuclear **OCNGS RADIOLOGICAL SURVEY**

No. RFC-95-1064 | Date 3-1-95 | Time 1030 | Location CLEAN UP VALVE ASSY. & CONTROL AREA

RWP 950001 | Reason Inspect V-16-103, V-2-715

Rx Power - 100 %



SMEARABLE CONTAMINATION

	LOCATION	BY ☐ CPM ☒ DPM ☒ MRAD/HR	CPM/DPM	AREA
1	Floor	300 K DPM	NC	100 cm²
2	Floor	140 K DPM	NC	"
3	Floor	9 MRAD	NC	"
4	V-2-715	80 K DPM	41	"
5	V-16-103	40 K ↑	32	"
6	o/s Piping	60 K	32	"
7	Light Fixture	445 ↓	NC	↓
8	o/s Piping	82 K DPM	NC	100 cm²
9				
10				
11				
12				
13				
14				
15				
16				
17				
18				
19				
20				

Surveyor: (print name) J. Lockhart

Signature J. Lockhart Date 3-1-95

Reviewer: (print name) T. Green

Signature T. Green Date 3-1-95

(#) Contact Dose Rates are Circled | [#] Smear Locations are Boxed

All Dose Rates are General Area Readings in mr/hr Unless Otherwise Noted

☒ No BETA Detected Unless Otherwise Noted ☐ No BETA Readings Ta

Remarks: V-2-715 Has bolt to bonnet leak, located 3' 10" w Overhead
V-16-103 located 5' 6" w overhead
1-10', 1-6' Strobubblers located in Room
Overhead Piping, very dusty, lightning is out.
A/S # 1318-95 - 2.5E-9 µCi

INSTRUMENTATION D

RADIATION SURVE
INST RO2A
S/N 4124 BCF
CDD 4-15-95

INST
S/N BCF
CDD

CONTAMINATION SU
INST RM-14
S/N 431
CDD 5-8-95

EFF 10% BKG 60
INST Ludlum 1000
S/N 22163
CDD 8-8-95

CF 3.2 BKG

AIR SAMPLE DAT
FC #

NOTES

NC = Not Counted
GW = Gross Wipe
NA = Not Applicable
NT = Not Taken
H = Head/C = Chest/T = T

G/41

N5268 105-

— Copy —

ALLEGATION DISPOSITION RECORD Rev. 2 10/5/94

Site: Oyster Creek Section Chief (AOC): Rogge

Allegation No.: RI-95-A-0026 Date Received: 2/6/95

Acknowledged: NRC will acknowledge Receipt Report to SAC: 2/6/95

CONFIDENTIALITY GRANTED: Yes (No) OI Informed: yes

IS THERE A HARASSMENT/DISCRIMINATION ISSUE: (If yes, complete H&ID section on reverse)	Yes	No
DOES THE ALLEGATION INVOLVE POTENTIAL WRONGDOING:	Yes	No ?
DOES THE ALLEGATION HAVE POLITICAL IMPLICATIONS:	Yes	(No)
DOES THE ALLEGATION REQUIRE RESOURCES TO RESOLVE WHICH CAN NOT BE OBTAINED BY THE AOC:	Yes	No

If yes to any of the above, the allegation needs to go to an Allegation Panel. Otherwise, document disposition actions below.

ALLEGATION PANEL (AP) DECISIONS

Date: 2/8/95 Previous APs on issue: Yes / (No)

Chair - Cooper NRR Branch Chief - Durr

Section Chief (AOC) - Rogge SAC - Vito

(Others) - G. Smith OI Rep. - Letts

DISPOSITION ACTIONS: (State specific action required for closure (including special concurrences), responsible person, ECD and expected closure documentation)

1) SAC will review the NRC acknowledgement letter to determine whether any additional info may need to be provided to alleger (e.g. notifying him of referral to licensee, or providing OI info if necessary)

Responsible Person: SAC ECD: TBD

Closure Documentation: _____ Completed: _____

2) Refer issues to licensee

Responsible Person: DRP/Rogge ECD: 2/28/95

Closure Documentation: _____ Completed: _____

3) DRSS to review licensee response - then Repanel issue

Responsible Person: DRSS/Smith ECD: TBD

Closure Documentation: _____ Completed: _____

4) No specific indication of wrongdoing at this time (per B. Letts)

Responsible Person: _____ ECD: 6/4/95

Closure Documentation: _____ Completed: _____

Safety Significance Assessment: low - individual issues of low significance - if subsequent assessment of issues collectively raises additional concerns, will reassess significance at that time