



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 26, 1996

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: ACKNOWLEDGMENT OF RECEIPT OF PRAIRIE ISLAND STEAM GENERATOR
INSPECTION REPORT AND REQUEST FOR ADDITIONAL INFORMATION

Dear Mr. Anderson:

This letter acknowledges receipt of your steam generator inspection report submittal dated February 28, 1996. Thank you for providing additional information regarding steam generator tube sleeves and for your cooperation in our meeting with Commonwealth Edison Company and ABB Combustion Engineering on March 6, 1996.

In your February 28, 1996, letter you made several commitments to the NRC. The NRC acknowledges your commitments (1) to pursue formal qualification of the eddy current examination technique used in determining whether to plug tubes with upper sleeve weld indications (i.e., volumetric) based on their locations in the weld joint; (2) to begin a mid-cycle shutdown for inspection of installed sleeves no later than eight operating months following the startup of Unit 1; (3) to evaluate the results of the metallurgical analysis to determine whether a Unit 1 mid-cycle inspection of the sleeves is warranted; and (4) to submit the results of the metallurgical examination and your conclusions to the NRC for review and concurrence within 90 days following Unit 1 startup.

During the course of the NRC staff review of your February 28, 1996, report, several areas requiring additional information or clarification were identified. The staff requests that you address the following comments in the report that is to be submitted to the NRC within 90 days following Unit 1 startup.

1. Discuss the nature of the inside diameter single axial indication 2 inches above the first support plate on the cold leg side of steam generator No. 11 (refer to page 1 of Attachment 1). Discuss your plans for addressing the root cause of this indication. Discuss industry experience, if known, with this form of degradation.
2. Discuss the nature of the "free span and top of tubesheet indications" cited on page 5 of Attachment 2. In addition, clarify the nature of the two free span crack-like indications reported on page 6 of Attachment 5. Discuss the root cause of these indications.
3. Clarify the use of the UT-360 System in dispositioning volumetric indications in sleeve welds. That is, clarify if depth sizing of indications was performed only for indications which (1) were acceptable by visual examination; (2) were acceptable by installation UT examination;

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(3) were acceptable by bobbin coil inspection; (4) were physically located at the upper edge of the weld; (5) were located in a weld with an average weld height of greater than 0.080 inches; and (6) were volumetric in nature.

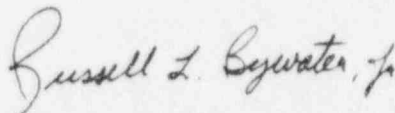
If depth sizing was performed for other indications, provide the depth sizing qualification data for the UT-360 System.

4. Clarify what is meant by the minimum average weld height (when referring to the height of the weld). Provide examples of how this quantity was calculated.
5. Provide a summary of the procedure and your basis for the use of eddy current examination to determine the distance from the actual bottom of the weld to the actual bottom of the volumetric indication.

The requested response affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under Public Law 96-511.

If you have any questions regarding these issues, please contact me at (301) 415-1355.

Sincerely,



Russell L. Bywater, Acting Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 & 50-306

cc: see next page

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Original Signed By:

Russell L. Bywater, Acting Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 & 50-306

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Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

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