



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TOLEDO EDISON COMPANY

AND

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. NPF-3

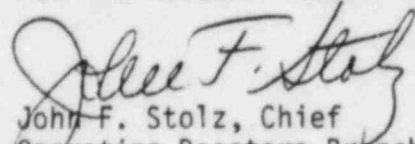
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Toledo Edison Company and The Cleveland Electric Illuminating Company (the licensees) dated February 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78, are hereby incorporated in the license. The Toledo Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 28, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 78

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

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TABLE 3.3-10
POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. SG Outlet Steam Pressure	1/Steam Generator
2. RC Loop Outlet Temperature	2/Loop
3. RC Loop Pressure	2/Loop
4. Pressurizer Level	2
5. SG Startup Range Level	2/Steam Generator
6. Auxiliary Feedwater Status	1/AFW System
7. Containment Vessel Hydrogen	2
8. Containment Vessel Post-Accident Radiation	2
9. Containment Vessel Isolation Status	1/Valve
10. SFAS Status	1/Channel
11. Safety Features Equipment Status	1/System
12. RPS Status	1/Channel
13. SFRCS Status	1/Channel
14. High Pressure Injection Flow	1/Channel

TABLE 3.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>FUNCTION CHANNELS OPERABLE</u>
15. Low Pressure Injection (LPI) Flow	1/Channel
16. HPI System Pump and Valve Status	1/System
17. LPI System Pump and Valve Status	1/System
18. Containment Spray Pump and Valve Status	1/System
19. Core Flood Valve Status	1/System
20. BWST Valve Status	1/System
21. Containment Emergency Sump Valve Status	1/Valve
22. Deleted	
23. Containment Air Cooling Fan Status	1/Fan
24. EVS Fan and Damper Status	1/System
25. Auxiliary Feedwater Flow Rate	2/Steam Generator
26. RC System Subcooling Margin Monitor	1
27. PORV Position Indicator	1
28. PORV Block Valve Position Indicator	1
29. Safety Valve Position Indicator	1/Valve
30. BWST Level	3

TABLE 4.3-10
POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. SG Outlet Steam Pressure	M	R*
2. RC Loop Outlet Temperature	M	R
3. RC Loop Pressure	M	R
4. Pressurizer Level	M	R
5. SG Startup Range Level	M	R
6. Auxillary Feedwater Status	M	NA
7. Containment Vessel Hydrogen	M	R
8. Containment Vessel Post-Accident Radiation	M	R
9. Containment Vessel Isolation Status	M	NA
10. SFAS Status	M	NA
11. Safety Features Equipment Status	M	NA
12. RPS Status	M	NA
13. SFRCS Status	M	NA
14. High Pressure Injection Flow	M	R

*18 month surveillance test due May 17, 1983, may be delayed until 2400 hours September 17, 1983.

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TABLE 4.3-10 (Continued)

POST-ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
15. Low Pressure Injection (DIIR) Flow	H	R
16. HPI System Pump and Valve Status	H	NA
17. LPI System Pump and Valve Status	H	NA
18. Containment Spray Pump and Valve Status	H	NA
19. Core Flood Valve Status	H	NA
20. BWST Valve Status	H	NA
21. Containment Emergency Sump Valve Status	H	NA
22. Deleted		
23. Containment Air Cooling Fan Status	H	NA
24. FVS Fan and Damper Status	H	NA
25. Auxiliary Feedwater Flow Rate	H	R
26. RC System Subcooling Margin Indicator	H	R
27. PORV Position Indicator	H	R
28. PORV Block Valve Position Indicator	H	R
29. Pressurizer Safety Valve Position Indicator	H	R
30. BWST Level	S	R

DAVIS BESSIE, UNIT 1

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Amendment 10, 11, 12, 13, 14, 15, 16, 17, 18, 19, 20, 21, 22, 23, 24, 25, 26, 27, 28, 29, 30, 31, 32, 33, 34, 35, 36, 37, 38, 39, 40, 41, 42, 43, 44, 45, 46, 47, 48, 49, 50, 51, 52, 53, 54, 55, 56, 57, 58, 59, 60, 61, 62, 63, 64, 65, 66, 67, 68, 69, 70, 71, 72, 73, 74, 75, 76, 77, 78, 79, 80, 81, 82, 83, 84, 85, 86, 87, 88, 89, 90, 91, 92, 93, 94, 95, 96, 97, 98, 99, 100