

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PALISADES NUCLEAR PLANT										DOCKET NUMBER (2) 0 5 0 0 0 0 2 5					PAGE (3) 1 OF 0 2	
TITLE (4) Primary Coolant System Unidentified Leakage Greater Than Limit																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
									NA				0 5 0 0 0			
1 1	1 5	8 4	8 4	0 2 4	0 0	1 2	1 7	8 4	NA				0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
N		20.402(b)				20.406(e)				80.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.406(a)(1)(i)				80.38(a)(1)				80.73(a)(2)(v)				73.71(a)		
0 0 1 0		20.406(a)(1)(ii)				80.38(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 305A)		
		20.406(a)(1)(iii)				X 80.73(a)(2)(i)				80.73(a)(2)(vii)(A)						
		20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)						
		20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
David W. Rogers; Technical Engineer; Palisades										AREA CODE 6 1 6 7 6 4 - 8 9 1 3						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
X	A A S E A	L G O 4 0		Y												
SUPPLEMENTAL REPORT EXPECTED (14)																
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO						
										EXPECTED SUBMISSION DATE (15)						
										MONTH DAY YEAR						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 15, 1984, with the Plant in hot shutdown, a leak rate test indicated primary coolant system unidentified leakage to be in excess of Technical Specifications limits. In accordance with Technical Specifications requirements, a cold shutdown condition was established within 24 hours.

Investigation determined that a control rod drive mechanism seal housing had failed. An engineering evaluation has determined that the failure is an isolated event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 5/31/85

FACILITY NAME (1) PALISADES NUCLEAR PLANT	DOCKET NUMBER (2) 0 5 0 0 0 2 5 5 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			0 2 4	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 15, 1984, with the Plant in hot shutdown, a four hour duration primary coolant system [AB] leak rate test indicated that unidentified leakage was in excess of the Technical Specification 3.1.5a limit. Calculations showed that unidentified leakage was 4.65 gallons per minute. The Technical Specifications limit for unidentified leakage is one gallon per minute.

Subsequent investigation identified abnormal temperatures on the #1 control rod drive mechanism [AA] and the presence of steam in the control rod drive leak-off. The #1 control rod drive mechanism (CRDM) was exercised and an increase in the leak rate was noted. A cooldown was initiated, and the Plant was placed in cold shutdown within 24 hours, as specified in Technical Specifications 3.1.5a.

Repair of the #1 CRDM was initiated and a failed seal housing [AA, SEAL] was identified. The seal housing was replaced.

An engineering evaluation was performed to identify the cause of the seal failure. Since the damage to the seal was extensive, no specific cause could be determined other than the entry of foreign material into the seal faces. No evidence of other CRDM seal problems was indicated, and no problems have occurred since the subsequent plant startup. The seal failure does not have generic implications and is considered an isolated occurrence.

Although the leak rate exceeded Technical Specifications limits, no threat to public health or safety resulted. The source of the leak was easily identified, and a safe and orderly shutdown was performed within Technical Specifications time limits.



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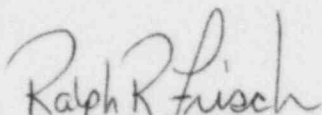
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December 17, 1984

US Nuclear Regulatory Commission
Document Control Desk Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-024 (PRIMARY COOLANT SYSTEM
UNIDENTIFIED LEAKAGE GREATER THAN LIMIT)

Attached please find Licensee Event Report 84-024 (Primary Coolant System
Unidentified Leakage Greater Than Limit) which is reportable to the NRC per 10
CFR 50.73(a)(2)(1).


Ralph R Frisch

Senior Licensing Analyst

CC Administrator, Region III, USNRC
Director, Office of Nuclear Reactor Regulation
NRC Resident Inspector - Palisades

Attachment