



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

June 18, 1992

Docket No. 50-302

Mr. Percy M. Beard, Jr.  
Senior Vice President,  
Nuclear Operations  
Florida Power Corporation  
ATTN: Manager, Nuclear Operations  
Licensing  
P.O. Box 219-NA-2I  
Crystal River, Florida 32629

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER UNIT 3 - NRC BULLETIN 88-08 "THERMAL STRESS IN PIPING  
CONNECTED TO REACTOR COOLANT SYSTEMS" (TAC NO. M69621)

By letters dated August 30, 1988 and September 13, 1989, you responded to the subject Bulletin. Our letter of February 6, 1992 found these responses, as we understood them, not entirely satisfactory and requested additional information and justification. Your letter of May 26, 1992 clarified your earlier responses. As we understand your collective responses, you had reviewed reactor coolant system (RCS) interfaces in response to a similar event at Crystal River 3 (CR-3) in 1982 involving the thermal sleeves at the high pressure injection/makeup (HPI/MU) lines. You again reviewed RCS interfaces in response to Supplement 3 of the Bulletin and verified that the potential for adverse thermal effects is limited to the HPI/MU lines. This addresses Action Item 1 of the Bulletin.

In addressing the 1982 problem at the time, you had examined the HPI/MU nozzles using radiography, dye penetrant, and ultrasonic techniques and found no existing flaws. This was not repeated in response to the Bulletin. This addresses Action Item 2 of the Bulletin.

In response to the 1982 event, the HPI/MU nozzles were redesigned, i.e., the thermal sleeves were lengthened and rerolled, and the upstream check valves were relocated further upstream. An analysis was performed to qualify the revised configuration. The piping arrangement was instrumented, monitored and analyzed to verify the design. An augmented inservice inspection plan was established to confirm the integrity of the nozzle and thermal sleeve using radiography during selected refueling outages. This addresses Action Item 3 of the Bulletin. Your earlier actions appear to moot the schedular requirements of Action Item 4 of the Bulletin.

Based on a re-review of your three submittals in response to Bulletin 88-08, we have determined that your responses are consistent with the actions stated in the Bulletin. Documentation which formed the basis for your responses to the Bulletin should be maintained in suitable form for possible future NRC audit.

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With regard to the enclosure to our February 6, 1992 letter, it should be noted that the "evaluation criteria" contained therein are, as stated in that letter, intended only to assist in preparation of acceptable responses. That enclosure does not consider differences in the design of individual plants or in the review, examination, or analyses performed by individual licensees to address the Bulletin and its supplements. It does not establish minimum criteria or preclude other means of satisfying the Bulletin.

This completes our efforts on TAC No. M69621.

Sincerely,

(Original Signed By)

Harley Silver, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

cc: See next page

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Generating Plant

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