

L I C E N S E E E V E N T R E P O R T ( L E R )

FACILITY NAME (1) Arkansas Nuclear One, Unit 2 DOCKET NUMBER (2) PAGE (3)  
10151010101 31 61 8110F1011  
TITLE (4) Reactor Trip Due To Low Steam Generator Water Level During Turbine Stop Valve Testing

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
Month	Day	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
11	1	01	3	81	4	11	1	01	01

OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10)	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.36(c)(1)	50.36(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(x)	73.71(b)	73.71(c)	Other (Specify in Abstract below and in Text, NRC Form 366A)	
													X									

LICENSEE CONTACT FOR THIS LER (12)

Name	Telephone Number
Patrick C. Rogers, Plant Licensing Engineer	Area Code 5101191614-1311010

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS
X	T	G	0101013	G101810	N				

SUPPLEMENT REPORT EXPECTED (14)

Yes (If yes, complete Expected Submission Date)  No

EXPECTED SUBMISSION DATE (15)

Month	Day	Year

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 11/3/84, at 1104 hours while in Mode 1 at 90% full power, an automatic reactor trip occurred due to a low water level in the "A" steam generator. Turbine stop valve surveillance testing was in progress at the time of the occurrence. During the closing stroke of stop valve #2, stop valves 1, 3 and 4 began closing. The subsequent "A" steam generator pressure increase resulted in a "shrink" in the "A" steam generator water level. The water level of "A" steam generator decreased to the actuation point for reactor trip and emergency feedwater start. No post trip operational difficulties were encountered. The interlock circuit for stop valve #2 was checked and no problems were found. Additional stop valve testing was performed with no subsequent failure. It is suspected that a mercury-wetted relay used for turbine stop valve test interlock failed to function properly. The relay card had been replaced due to an unrelated problem which occurred during a plant startup on 10/31/84. The replacement card had been in storage and it is suspected that the mercury-wetted contacts remained wetted after installation. This initial operation apparently restored the mercury to its reservoir and subsequent checkout failed to identify a repetitive problem with the relay. There have been no previous reportable occurrences of reactor trip on low steam generator water level due to turbine stop valve testing.

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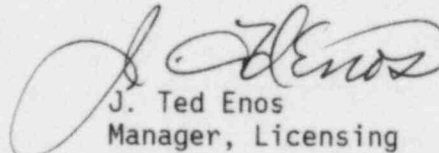
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Subject: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Licensee Event Report  
No. 84-028-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning the occurrence of an automatic reactor trip due to a low water level in the "A" steam generator.

Very truly yours,

  
J. Ted Enos  
Manager, Licensing

JTE:RJS:ds

Attachment

cc: Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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