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Docket No. 50-333
EA No. 92-033

New York Power Authority
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. Harry P. Salmon, Jr.
Resident Manager
Post Office Box 41
Lycoming, New York 13093

Dear Mr. Salmon:

SUBJECT: NRC SAFETY SYSTEM FUNCTIONAL INSPECTION OF THE
EMERGENCY SERVICE WATER SYSTEM, NRC REGION I INSPECTION
REPORT NO. 50-333/92-81

This refers to the Safety System Functional Inspection team inspection conducted by Mr. James Trapp and other NRC personnel on April 13 through May 1, 1992. The inspection included a review of activities authorized for your James A. FitzPatrick facility, in Scriba, New York. At the conclusion of the inspection, the team findings were discussed with you, and members of your staff, on May 1, 1992. The primary purpose of this inspection was to determine whether the emergency service water system was designed, and is being operated, tested and maintained such that it would be capable of performing its intended safety function.

Areas examined during the inspection are discussed in the enclosed inspection report. Within these areas, the inspection consisted of selective examinations of procedures, design calculations, installed equipment, interviews with personnel, and observations by the inspectors. The emergency service water system is important to safety because it transfers heat from safety-related equipment used for mitigating accidents to Lake Ontario. Based on the information reviewed, the inspection team concluded that overall the emergency service water system is capable of performing its intended function; however, the team had two unresolved items regarding the crescent area unit coolers, where additional testing and analysis are required to verify that these coolers can perform their design function.

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The team determined that a significant effort had been made to improve the emergency service water system. These improvements included several modifications which improved the system performance. Other identified strengths were the validation of the design basis document and the quarterly integrated flow test of the ESW system. The team also identified several weaknesses such as an inadequate safety evaluation for downgrading the control room chiller condensers to non-safety related, and inadequate emergency service water flow to the crescent area unit coolers. Additional concerns regarding the heat transfer testing, and the design adequacy of the crescent area unit coolers were left unresolved.

A deviation from the final safety analysis report (FSAR) minimum required service water flow to each crescent area unit coolers was identified. On a number of occasions in 1991, crescent area unit coolers were provided less than the FSAR required minimum flow. Providing adequate flow to the crescent area unit coolers is important to assure that safety-related equipment located in the crescent area will function if called upon to operate. You are requested to respond to the enclosed Notice of Deviation.

Based on the results of this inspection, one of your activities appears to be in violation of NRC requirements, as specified in the enclosed Notice of Violation (Notice). The violation is the failure to perform an adequate safety evaluation when downgrading the control room chiller condensers from safety-related to non-safety related. Corrective actions were completed during the inspection to reduce the maximum control room temperature during a design-basis accident, to within the original design specification. Technically sound safety evaluations are important to assure that modifications to safety-related components do not reduce the original margin of safety provided in the design-basis.

You are required to respond to this letter and should follow the instructions specified in the enclosed Notice when preparing your response. In your response, you should document the specific actions taken and any additional actions you plan to prevent recurrence. After reviewing your response to this Notice, including your proposed corrective actions and the results of future inspections, the NRC will determine whether further NRC enforcement action is necessary to ensure compliance with NRC regulatory requirements.

A second violation of NRC requirements was identified regarding the failure to conduct inservice surveillance tests on manual valves 70WAC-5A and 5B, in accordance with the American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section XI. The violation is not being cited because the criteria specified in Section V.A of the Enforcement Policy were satisfied.

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In addition, one apparent violation was identified and is being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions" (Enforcement Policy), 10CFR Part 2, Appendix C. Incomplete or inaccurate information was identified in five separate correspondences with the NRC. Accordingly, no Notice of Violation is presently being issued for these inspection findings. Please be advised that the number and characterization of apparent violations described in the enclosed inspection report may change as a result of further NRC review.

An enforcement conference to discuss this apparent violation has been scheduled for June 24, 1992. If after your review you find that you have problems meeting that date, please contact me. The purposes of this conference are to discuss the apparent violation(s), its (their) cause(s) and safety significance; to provide you the opportunity to point out any errors in our inspection report; to provide an opportunity for you to present your proposed corrective action; and to discuss any other information that will help us determine the appropriate enforcement action in accordance with the Enforcement Policy. You will be advised by separate correspondence of the results of our deliberations on this matter. No response regarding the apparent violation is required at this time.

Sincerely,

Original Signed By:
Marvin W. Hodges

Marvin W. Hodges, Director
Division of Reactor Safety

Enclosure: NRC Region I Inspection Report Number 50-333/92-81

cc w/encl:

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Region I Docket Room (with concurrences)

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