NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Limerick Generating Station - Unit 1	0 5 0 0 0 3 5 2	1 OF 0 3

TITLE (4)
Actuation of Emergency Core Cooling System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 1	1 4	8 4	8 4	- 0 0 7 -	0 0	1 2	1 4	8 4		0 5 0 0 0
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OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
5	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
B. L. Clark, Senior Engineer-Special Projects	2 1 5 8 4 1 - 5 0 1 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 84-007

On November 14, 1984, prior to initial criticality, the 'C' core spray pump, 'C' RHR pump, and D13 diesel generator automatically started as a result of receiving an inadvertent reactor low level signal. Additionally, Division III valves of the Nuclear Steam Supply Shutoff System isolated as a result of the same reactor low level signal. Cause of the event was improper venting of a reactor level instrument rack. All systems operated as designed. After the low level signal was cleared, the affected systems were returned to normal operation.

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Description of the Event:

On November 14, 1984 at 12:51 p.m., prior to initial criticality, 'C' core spray pump, 'C' RHR pump and D13 diesel generator automatically started as a result of an inadvertent reactor low level signal. Additionally, Division III of the Nuclear Steam Supply Shutoff System received a reactor low level isolation signal. The 'C' core spray pump and the 'C' RHR pump operated on minimum flow recirculation to the suppression pool. D13 diesel generator output breaker did not close onto its emergency bus, since the bus was already energized. Division III valves of the Nuclear Steam Supply Shutoff System isolated properly. After the low level signal was cleared, the affected systems were returned to normal operation.

Consequences of the Event:

The portions of the Emergency Core Cooling System and Nuclear Steam Supply Shutoff System that were activated by the inadvertent reactor low level signal functioned properly. There were no adverse consequences.

Cause of the Event:

Reactor level transmitter LT-42-IN080C was being replaced by Maintenance personnel. After the instrument was replaced and valved back into service, the half-scram would not clear. Instrument technicians were requested to vent the low pressure side of LT-42-IN080C to clear the half-scram. The low pressure side of LT-42-IN080C is common to the low pressure sides of devices LT-42-IN091C, LT-42-IN091G, and LT-42-IN081C. While venting the low pressure side of LT-42-IN080C, a pressure spike caused LT-42-IN091C, LT-42-IN091G, and LT-42-IN081C to actuate their respective switches. Actuating LT-42-IN091C and LT-42-IN091G simultaneously causes the 'C' core spray pump, 'C' RHR pump, and D13 diesel generator to automatically start. Actuating LT-42-IN081C causes a Division III isolation of the Nuclear Steam Supply Shutoff System. Cause of the event was improper venting of LT-42-IN080C.

Corrective Actions:

Valve information tags cautioning workers to contact shift supervision prior to operating the instrument valves were in place at the time of the event. The instrument technicians involved were counseled on the importance of complying with these information tags.

In addition, the station has requested that a modification to install head chambers on safety-related instruments be expedited to assist in backfilling the instruments after maintenance or calibration.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

December 14, 1984

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the inadvertent reactor low level signal that initiated a portion of the Emergency Core Cooling System and the Nuclear Steam Supply Shutoff System. This event occurred prior to initial criticality.

Reference:	Docket 50-352
Report Number:	84-007
Revision Number:	00
Event Date:	November 14, 1984
Report Date:	December 14, 1984
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(IV).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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cc: Judge Helen F. Hoyt
Judge Jerry Harbour
Judge Richard F. Cole
Judge Christine N. Kohl
Judge Gary J. Edles
Judge Reginald L. Gotchy
Troy B. Conner, Jr., Esq.
Ann P. Hodgdon, Esq.
Mr. Frank R. Romano
Mr. Robert L. Anthony
Ms. Phyllis Zitner
Charles W. Elliott, Esq.
Zori G. Ferkin, Esq.
Mr. Thomas Gerusky
Director, Penna. Emergency Management Agency
Angus Love, Esq.
David Wersan, Esq.
Robert J. Sugarman, Esq.
Martha W. Bush, Esq.
Spence W. Perry, Esq.
Jay M. Gutierrez, Esq.
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James Wiggins
Timothy R. S. Campbell