

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 050003611	PAGE (3) 1 OF 02
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TITLE (4)
CONTAINMENT PURGE ISOLATION SYSTEM (CPIS) ACTUATIONS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
11	09	84	84	069	00	12	07	84			05000

OPERATING MODE (9) 6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 000	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	AREA CODE	NUMBER	
J. G. HAYNES, STATION MANAGER	714	492-1710	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 11/9/84, at 1536, with Unit 2 in Mode 6, a Containment Purge Isolation System (CPIS) actuation occurred. The actuation occurred when bagged radioactive waste was transported past Containment Area Radiation Monitor 2RI-7856. Subsequent to this date, two additional CPIS actuations occurred on 1/13. These actuations, initiated by Containment Area Radiation Monitor 2RI-7857, were apparently caused by radioactive items being moved in the vicinity of the monitor. Monitors 2RI-7856 and 2RI-7857 have setpoints of less than or equal to 325 mR/hr in Modes 1-4, but the setpoints are reduced to less than or equal to 2.4 mR/hr in Mode 6. All CPIS components actuated properly on each occurrence.

Corrective action has been taken to prevent radioactive items from being placed in the vicinity of Monitors RI-7856 and RI-7857, and appropriate personnel have been informed to notify Operations that a CPIS actuation may occur when movement of radioactive items in the vicinity of these monitors cannot be avoided. Procedures are also being reviewed to identify other evolutions that may cause similar events and appropriate revisions will be made.

There are no reasonable or credible circumstances which could have increased the severity of these events. Neither the health and safety of plant personnel nor the public were affected.

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1 1	LER NUMBER (6)			PAGE (3)	
		YEAR 8 4	SEQ. NUMBER - 0 6 9	REV. NUMBER - 0 0		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 9, 1984, at 1536, with Unit 2 in Mode 6, a Containment Purge Isolation System (CPIS) (EIS System Code VA) actuation occurred. Subsequent to this date, two additional CPIS actuations occurred on November 13. All CPIS components actuated properly on each occurrence.

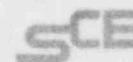
The November 9 actuation occurred when bagged radioactive waste was transported past Containment Area Radiation Monitor RI-7856 (EIS Component Code RIT). The bag contained removed Incore Instrumentation and was being carried from a container inside containment to another container outside of containment. The setpoint of Monitor RI-7856 had been reduced from less than or equal to 325 mR/hr to less than or equal to 2.4 mR/hr in accordance with Technical Specification requirements for Mode 6. Radiation levels at the monitor exceeded the lowered setpoint as the bag was carried past the monitor.

The first November 13 actuation occurred at the same time that the Upper Guide Structure was removed from the Reactor Vessel. With water at the refueling level, the top of the Incore Instrumentation Clusters is lifted out of the water when the Upper Guide Structure is lifted out of the Reactor Vessel. This may have caused the actuation. The second actuation occurred while several activities involving radioactive items were ongoing, and it is not known which specific item, if any, caused the actuation. Both of these actuations were initiated by Monitor RI-7857, which also had its setpoint reduced from less than or equal to 325 mR/hr to less than or equal to 2.4 mR/hr. A Containment Main Purge was in progress at the time of both actuations, and was properly isolated on each occurrence.

Corrective action has been taken to prevent radioactive items from being placed in the vicinity of Monitors RI-7856 and RI-7857 by placing a rope around the monitors, and appropriate personnel have been informed to notify Operations that a CPIS actuation may occur when movement of radioactive items in the vicinity of these monitors cannot be avoided. Procedures are being reviewed to identify other evolutions that may cause similar events and appropriate revisions will be made.

There are no reasonable or credible circumstances which could have increased the severity of these events. Neither the health and safety of plant personnel nor the public were affected.

Southern California Edison Company



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SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

December 7, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 84-069
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for three occurrences involving the actuation of the Containment Purge Isolation System. Neither the health and safety of plant personnel nor the public were affected by these events.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 84-069

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

Institute of Nuclear Power Operations (INPO)

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