



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95  
License No. NPF-57

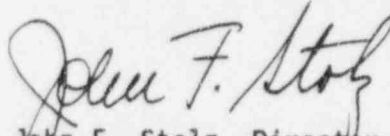
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company (PSE&G) dated April 13, 1994, as supplemented December 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-57 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9<sup>5</sup>, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the license. PSE&G shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: March 11, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 95

FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
6-8	6-8
6-11	6-11
6-12	6-12
6-15	6-15

- procedures that require a 10 CFR 50.59 safety evaluation as described in Section 6.5.3.2.d.
- b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to Appendix "A" Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Review of the safety evaluations that have been completed under the provisions of 10 CFR 50.59.
  - f. Initiation or review of investigations of all violations of the Technical Specifications including the reports covering evaluations and recommendations to prevent recurrence.
  - g. Review of all REPORTABLE EVENTS.
  - h. Review of facility operations to detect potential nuclear safety hazards.
  - i. Performance of special reviews, investigations or analyses and reports thereon as determined by SORC.
  - j. Not used.
  - k. Not used.
  - l. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10 CFR 50.59 safety evaluation.
  - m. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.
  - n. Review of changes to the PROCESS CONTROL MANUAL and the OFF-SITE DOSE CALCULATION MANUAL.

REVIEW PROCESS

6.5.1.7 A technical review and control system utilizing qualified reviewers shall function to perform the periodic or routine review of procedures and changes thereto. Details of this technical review process are provided in Section 6.5.3.

- f. All REPORTABLE EVENTS.
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- h. Reports and meeting minutes of the SORC.

AUDITS

6.5.2.4.3 Audits of facility activities shall be performed under the cognizance of the OSR staff. These audits shall encompass the following:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50.
- e. Not used.
- f. Not used.
- g. Any other area of facility operation considered appropriate by the Director - Quality Assurance and Nuclear Safety Review or the Chief Nuclear Officer and President - Nuclear Business Unit.
- h. The facility Fire Protection Program and the implementing procedures.
- i. An audit of the fire protection and loss prevention program implementation utilizing an outside independent fire protection consultant.
- j. The radiological environmental monitoring program and the results thereof.
- k. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.

1. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.
- m. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring.

The above audits shall be conducted by the Quality Assurance Department or an independent consultant. Audit plans and final audit reports shall be reviewed by the OSR staff.

#### RECORDS AND REPORTS

6.5.2.4.4 Records of OSR activities shall be maintained. Reports of reviews and audits shall be prepared and distributed as indicated below:

- a. The results of reviews performed pursuant to Section 6.5.2.4.3 shall be reported to the Chief Nuclear Officer and President - Nuclear Business Unit at least monthly.
- b. Audit reports prepared pursuant to Specification 6.5.2.4.3 shall be forwarded by the auditing organization to the Chief Nuclear Officer and President - Nuclear Business Unit and to the management positions responsible for the areas audited (1) within 30 days after completion of the audit for those audits conducted by the Nuclear Quality Assurance Department, and (2) within 60 days after completion of the audit for those audits conducted by an independent consultant.

#### 6.5.2.5 ONSITE SAFETY REVIEW GROUP (SRG)

##### FUNCTION

6.5.2.5.1 The SRG shall function to provide: the review of plant design and operating experience for potential opportunities to improve plant safety; evaluation of plant operations and maintenance activities; and advice to management on the overall quality and safety of plant operations.

The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

##### RESPONSIBILITIES

6.5.2.5.2 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics, NRC issuances, industry advisories, and other appropriate sources of plant design and operating experience information which may indicate areas for improving plant safety.
- b. Review of selected facility features, equipment, and systems.
- c. Review of selected procedures and plant activities including maintenance, modification, operational problems, and operational analysis.

- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the Director - Quality Assurance and Nuclear Safety Review and the Chief Nuclear Officer and President - Nuclear Business Unit within 30 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

#### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978.
- b. The applicable procedures required to implement the requirements of NUREG-0737 and supplements thereto.
- c. Refueling operations.
- d. Surveillance and test activities of safety-related equipment.
- e. Security Plan implementation.
- f. Emergency Plan implementation.
- g. Fire Protection Program implementation.
- h. PROCESS CONTROL PROGRAM implementation.
- i. OFFSITE DOSE CALCULATION MANUAL implementation.
- j. Quality Assurance Program for effluent and environment monitoring.

6.8.2 Each procedure and administrative policy of 6.8.1 above, except 6.8.1.e and 6.8.1.f, and changes thereto, shall be reviewed and approved in accordance with specification 6.5.1.6 or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.e and 6.8.1.f shall be reviewed and approved in accordance with the Facility's Security and Emergency Plans or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 On-the-Spot changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the unit management staff, at least one of whom holds a Senior Reactor Operator license on the unit affected; and