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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 19, 1984

Dr. Robert M. Lazo, Chairman
Administrative Judge
Atomic Safety and Licensing Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Richard F. Cole
Administrative Judge
Atomic Safety and Licensing Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dr. Emmeth A. Luebke
Administrative Judge
Atomic Safety and Licensing Board
U.S. Nuclear Regulatory Commission
Washington, DC 20555

In the Matter of
FLORIDA POWER AND LIGHT COMPANY
(Turkey Point Plant, Unit Nos. 3 and 4)
Docket Nos. 50-250, 50-251 OLA-2

Dear Administrative Judges:

Enclosed for your information is a copy of a letter dated December 13, 1984, from the NRC Staff transmitting corrected pages for Technical Specifications modified by recent amendments. A Technical Specification page issued with the amendment authorizing an expanded storage capacity for the two spent fuel pools (Amendments 111/105) is among the pages containing certain administrative errors. Copies of the corrections have already been sent to the parties.

Sincerely,

Mitzi A. Young

Mitzi A. Young
Counsel for NRC Staff

Enclosure: As stated

cc w/o enclosure: Service list

DS07

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 13, 1984

Docket Nos. 50-250
and 50-251

Mr. J. W. Williams, Jr., Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

Dear Mr. Williams:

During the period of August 27, 1984 through November 21, 1984, the Commission has issued six amendments to Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Units 3 and 4. Your staff has brought to our attention administrative errors on some of the Technical Specification pages. These typographical errors, inadvertent omissions or other errors were the result of issuing the amendments in a different sequence than they were requested. It is our understanding that a system has been initiated by your staff to track all active amendment requests to assure that administrative errors will be identified prior to issuance of amendments regardless of the request or issuance sequence. We will work closely with your staff to eliminate administrative errors prior to the issuance of future amendments.

Administrative errors are identified in the Technical Specification Table of Content pages i and iv, Technical Specification pages 1-9 and 3.1-4, Table 4.1-1 Sheets 3 and 4 and the Bases page B2.3-3. Amendments 110/104 left out the identification of Section 1.35 on page i and Amendments 111/105 left out the identification of B4.18 and B4.19 on page iv. Amendments 108/102 resulted in an incorrect sequence of numbers and Item 1.35 was omitted from page 1-9. Amendments 110/104 had a typographical error which resulted in the last sentence in paragraph f being partially omitted. Amendments 110/104 omitted Item 31.b and Item 33 on Table 4.1-1 Sheet 3 and an incorrect numbering sequence on Sheet 4. Amendments 106/100 resulted in incorrect wording of the second paragraph on page B2.3-3.

Enclosed are the corrected pages i, iv, 1-9, 3.1-4, Table 4.1-1 Sheets 3 and 4, and B2.3-3. We have verified that the errors do not affect the amendments, the supporting Safety Evaluations and the corrections are consistent with their applicable amendments. Please use the enclosed instruction sheet and replace the pages identified in Appendix A of Facility Operating License Nos. DPR-31 and DPR-41 with the corrected pages and Tables.

Please accept our apologies for any inconvenience these errors may have caused. As stated above, we will work closely with your staff to eliminate administrative errors in issuance of future amendments.

Sincerely,

Daniel G. McDonald Jr.

Daniel G. McDonald, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
As stated

cc w/enclosures:
See next page

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AMENDMENT CORRECTIONS
FACILITY LICENSE NOS. DPR-31 AND DPR-41
DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages	Insert Pages
i Amendment Nos. 110 and 104	i Amendment Nos. 110 and 104
iv Amendment Nos. 111 and 105	iv Amendment Nos. 111 and 105
1-9 Amendment Nos. 108 and 102	1-9 Amendment Nos. 108 and 102
3.1-4 Amendment Nos. 110 and 104	3.1-4 Amendment Nos. 110 and 104
Table 4.1-1 Sh 3 & Sh 4 Amendment Nos. 110 and 104	Table 4.1-1 Sh 3 & Sh 4 Amendment Nos. 110 and 104
B 2.3-3 Amendment Nos. 106 and 100	B 2.3-3 Amendment Nos. 106 and 100

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1.30 **PURGE - PURGING**

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

1.31 **VENTING**

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is not provided or required during VENTING. Vent, used in system names, does not imply a VENTING process.

1.32 **SITE BOUNDARY**

The SITE BOUNDARY shall be that line beyond which the land is neither owned, leased nor otherwise controlled by the licensee.

1.33 **UNRESTRICTED AREA**

An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation from radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional and/or recreational purposes.

1.34 **MEMBER(S) OF THE PUBLIC**

MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the licensee, its contractors, vendors or members of the Armed Forces using property located within the SITE BOUNDARY. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational or other purposes not associated with the plant.

1.35 **HEAVY LOADS**

Any load in excess of the nominal weight of a fuel and control rod assembly and associated handling tool. For the purpose of this specification, HEAVY LOADS will be defined as loads in excess of 2000 pounds.

e. After shutdown, corrective action shall be taken before operation is resumed.

f. Above 2% of rated power, two leak detection systems of different principles shall be operable, one of which is sensitive to radioactivity. The latter may be out of service for 48 hours provided 1) Two other systems are operable and 2) containment purge valves are maintained closed.

g. Reactor Coolant System leakage shall be limited to 1 gpm total primary-to-secondary leakage through all steam generators not isolated from the Reactor Coolant System and 500 gallons per day through any one steam generator not isolated from the Reactor Coolant System.

4. MAXIMUM REACTOR COOLANT ACTIVITY

The specific activity of the primary coolant shall be limited to:

a. Less than or equal to 1.0 microcurie per gram DOSE EQUIVALENT I-131, and

b. Less than or equal to 100/$\bar{E}$ microcuries per gram.

With the above limits being exceeded, the following actions shall be taken:

1. When the reactor is critical or average reactor coolant temperature is greater than 500 F:

a. With the specific activity of the primary coolant greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 but within the allowable limit (below and to the left of the line) shown on Figure 3.1-1, operation may continue for up to 48 hours provided that the cumulative operating time under these circumstances does not exceed 800 hours in any consecutive 12 month period. With the total cumulative operating time at a primary coolant specific activity greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 exceeding 500 hours in

	Channel Description	Check	Calibrate	Test	Remarks
23.	Logic Channels	N.A.	N.A.	M†	
24.	Emer. Portable Survey Instruments	N.A.	A	M	
25.	Seismograph	N.A.	N.A.	Q	Make trace. Test battery (change semi-annually)
26.	Auxiliary Feedwater Flow Rate	M†	R	N.A.	
27.	RCS Subcooling Margin Monitor	M†	R	N.A.	
28.	PORV Position Indicator (Primary Detector)	M†	N.A.	R	Check consists of monitoring indicated position and verifying by observation of related parameters.
29.	PORV Block Valve Position Indicator	M†	N.A.	R	
30.	Safety Valve Position Indicator	M†	R	N.A.	
31.	a. Loss of Voltage (both 4kv bussess)	N.A.	N.A.	R	For AFW actuation at power only
	b. Undervoltage (both 4KV busses and 480 volt Load Centers)**	S†	R	M†	
32.	Trip of both Main Feedwater Pump Breakers	N.A.	N.A.	R	For AFW actuation at power only
33.	Turbine Trip (Auto-Stop Oil Pressure Switches)	N.A.	R.	N.A.	

** This item does not apply on Unit 3 until after implementation of PC/M 79-116 and on Unit 4 until after implementation of PC/M 80-44.

TABLE 4.1-1 SHEET 4

	Channel Description	Check	Calibrate	Test	Remarks
34.	Containment Water Level (Narrow Range)	M††	R	N.A.	
35.	Containment Water Level (Wide Range)	M††	R	N.A.	
36.	Containment High Range Area Radiation	S††	R(Note 1)	M††	
37.	Containment Hydrogen Monitors	S†	Q(1)	M†	(1) Channel calibration using sample gas containing: a. One volume percent hydrogen, balance nitrogen. b. Four volume percent hydrogen, balance nitrogen.
38.	High Range Noble Gas Effluent Monitors				
	a. Plant Vent Exhaust	S	R	M	
	b. Unit 3 Spent Fuel Pit Exhaust	S	R	M	
	c. Condenser Air Ejectors	S†	R	M†	
	d. Main Steam Lines	S†	R	M†	
39.	Incore Thermocouples (Core Exit Thermocouples)	M††	R	N.A.(See Note 2)	

Amendment Nos. 110 and 104

Reactor Trip Interlocks

Specified reactor trips are by passed at low power where they are not required for protection and would otherwise interfere with normal operation. The prescribed set points above which these trips are made functional assures their availability in the power range where needed.

An automatic reactor trip will occur if any pump is lost above 55% power which will prevent the minimum value of the DNBR from going below the applicable design limit during normal and anticipated transient operations when only two loops are in service,(9) and the overtemperature ΔT trip setpoint is adjusted to the value specified for three loop operation.

A turbine trip initiates a reactor trip. On decreasing power, the turbine trip is automatically blocked by P-7; and on increasing power reinstated automatically by P-7.

Reset of reactor trip interlocks will be done under strict administrative control.

References

(1) FSAR 14.1.1
(2) FSAR 14.1.2
(3) FSAR 14.1
(4) FSAR 7.2, 7.3
(5) FSAR 3.2.1
(6) FSAR 14.3.1
(7) FSAR 14 (page 14-3 and 14.1.9)
(8) FSAR 14.1.11
(9) FSAR 14.1.9
(10) WCAP-8074