

NOTICE OF VIOLATION

Commonwealth Edison Company
LaSalle County Station, Unit 2

Docket Nos. 50-374
License No. NPF-18

During an NRC inspection conducted on April 20, 1992, through May 14, 1992, two violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1992), the violations are listed below:

1. 10 CFR 50, Appendix B, Section V states, in part, that activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances.

Contrary to the above, LaSalle Operating Procedure (LOP)RT-03, "Reactor Water Cleanup System Shutdown," Revision 7, was inappropriate to the circumstances given that the training of the operators did not require substeps to be done in sequence and, as such the procedure allowed operation of the system in a manner that resulted in reactor coolant leakage outside the primary containment.

This is a Severity Level IV violation (Supplement I).

2. LaSalle Technical Specification 6.2.A.1 requires, in part, that detailed written procedures covering items in Regulatory Guide 1.33, Appendix A, Revision 2, be prepared, approved, and adhered to. Regulatory Guide 1.33 lists procedures for the maintaining of containment integrity and administrative procedures.

The annunciator procedure LOA H13-P601 B507 requires the operator to identify and correct the high differential flow condition and if this cannot be achieved, to verify that automatic actions occur.

The station administrative procedure LAP-1600-2, "Conduct of Operations" Step F.1.au.5 states, "If a safety function initiates automatically assume a true initiating event has occurred unless otherwise confirmed by at least two independent indications."

Contrary to the above, on April 20, 1992, at approximately 8:50 a.m. the Unit 2 nuclear station operator bypassed a valid reactor water cleanup differential flow isolation signal, thereby failing to identify and correct the high differential flow condition and failing to assume that a true initiating event had occurred. The operators did not have instrumentation available to them to confirm that the signal was spurious.

This is a Severity Level IV violation (Supplement I).

JUN 1 1992

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington D.C. 20555, with a copy to the U.S. Nuclear Regulatory Commission, Region III, 799 Roosevelt Road, Glen Ellyn, Illinois, 60137, and a copy to the NRC Resident Inspector at the LaSalle facility, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or a demand for information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
this 1st day of June 1992


for Edward G. Greenman, Director
Division of Reactor Projects