



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 7, 1996

Mr. C. Randy Hutchinson
Vice President, Operations GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

SUBJECT: GENERIC LETTER 88-20, INDIVIDUAL PLANT EXAMINATION (IPE) -
INTERNAL EVENTS - GRAND GULF NUCLEAR STATION (TAC M74415)

Dear Mr. Hutchinson:

By letters dated December 23, 1992, and February 3, 1993, you submitted the IPE on severe accident vulnerabilities for internal events and internal flooding for the Grand Gulf Nuclear Station, Unit 1, (GGNS). Enclosed is the staff's safety evaluation for this IPE and the contractor's technical evaluation report (TER). The staff employed Sandia National Laboratory to review the IPE submittal.

The review of the IPE was based on a comparison between the results reported in the IPE and the results of the previous severe accident NUREG-1150 study of GGNS. Severe accidents at GGNS had previously been analyzed by the Nuclear Regulatory Commission (NRC) in NUREG-1150, "Severe Accident Risks: An Assessment for Five U.S. Nuclear Power Plants," dated June 1989.

The GGNS IPE was in response to Generic Letter (GL) 88-20, "Individual Plant Examination For Severe Accident Vulnerabilities," dated November 23, 1988, and its associated supplements. The staff has concluded that the IPE process documented in your submittal is capable of identifying the most likely severe accidents and severe accident vulnerabilities for GGNS and, therefore, that the GGNS IPE has met the intent of GL 88-20. The details of the staff's evaluation are in the enclosed safety evaluation and contractor's TER. This completes the work by the staff under TAC No. M74415.

It should be noted that the staff's review primarily focused on Entergy Operations, Inc.'s ability to examine GGNS for severe accident vulnerabilities. Therefore, the enclosed safety evaluation does not constitute NRC approval or endorsement of any IPE material for purposes other than those associated with meeting the intent of GL 88-20.

In accordance with GL 88-20, you proposed to resolve Unresolved Safety Issue (USI) A-45, "Shutdown Decay Heat Removal [DHR] Requirements." No other specific USIs or generic safety issues were proposed for resolution as part of the GGNS IPE. Based on the discussions of the core damage frequency (CDF) in the enclosed TER and that the total CDF for GGNS in the IPE is $2E-5$ /reactor year, the staff concludes that USI A-45 has been resolved for GGNS.

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In the IPE, you did not identify any plant vulnerabilities for GGNS; however, you identified the following plant improvements for implementation at the unit:

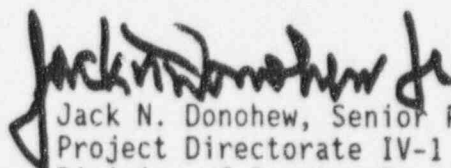
- Revise the offsite power off-normal event procedure to allow bypass of Level 2 signal in order to allow Division 3 power cross-tie with Divisions 1 and 2.
- Improve the procedure for bypassing the reactor core isolation cooling (RCIC) leak detection system trip when plant service water is unavailable and no steam leak has occurred.

and the following plant improvements for further consideration:

- Change containment isolation requirements to allow bypassing the containment isolation signal upon loss of Division 1 or 2 ac or dc bus and the re-opening of the containment isolation valves in order to retain the availability of plant service water and instrument air.
- Improve operator training on control room indication changes to standby service water (SSW) pump house ventilation status.
- Improve operator training on alternate operation of low-pressure emergency core cooling system pumps to minimize impact of SSW dependence.
- Modify the portion of the emergency operating procedures that directs the operators to vent the reactor vessel even if core damage has occurred.

When the above considerations have been completed, you are requested to inform the staff of the results of these considerations and what has been implemented at the plant. This request affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. 96-511. If you have any questions, please contact me at (301) 415-1307.

Sincerely,



Jack N. Donohew, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosures: 1. Safety Evaluation
2. Technical Evaluation Report

cc w/encls: See next page

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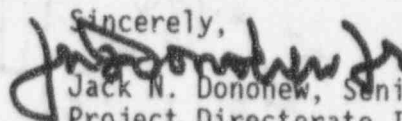
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OFC	LA:PD4-1	PM:PD4-1
NAME	PNoonan	JDonohew:sp
DATE	3/7/96	3/7/96
COPY	YES/NO	YES/NO

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