



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 181
License No. DPR-70

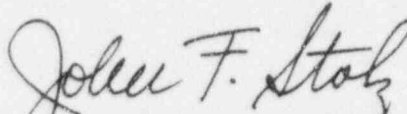
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated April 13, 1994, as supplemented December 6, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 181, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate 1-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 11, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 181

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

6-9

6-12

6-13

6-17

Insert Pages

6-9

6-12

6-13

6-17

ADMINISTRATIVE CONTROLS

QUORUM (continued)

shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of: (1) Upper tier administrative procedures within the scope of Regulatory Guide 1.33 (2/78), and changes thereto and (2) Newly created procedures or changes to existing procedures that require a 10CFR50.59 safety evaluation as described in Section 6.5.3.2.d.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review of the safety evaluations that have been completed under the provisions of 10CFR50.59.
- f. Investigation of all violations of the Technical Specifications including the reports covering evaluation and recommendations to prevent recurrence.
- g. Review of all REPORTABLE EVENTS.
- h. Review of facility operations to detect potential nuclear safety hazards.
- i. Performance of special reviews, investigations or analyses and reports thereon as requested by the General Manager - Salem Operations.
- j. Not used.
- k. Not used.
- l. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10CFR50.59 safety evaluation.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.4.2 The OSR staff shall review:

- a. The safety evaluations for changes to procedures, equipment, or systems; and tests or experiments completed under the provisions of 10CFR50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems, and tests or experiments that involve an unreviewed safety question as defined in 10CFR50.59;
- c. Proposed changes to Technical Specifications or to the Operating License;
- d. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- f. All REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Reports and meeting minutes of the Station Operations Review Committee.

AUDITS

6.5.2.4.3 Audits of facility activities shall be performed under the cognizance of the OSR staff. These audits shall encompass the following:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training, and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the Criteria of Appendix B to 10CFR50.

ADMINISTRATIVE CONTROLS

AUDITS (continued)

- e. Not used.
- f. Not used.
- g. Any other area of facility operation considered appropriate by the Director - Quality Assurance and Nuclear Safety Review or the Chief Nuclear Officer and President - Nuclear Business Unit.
- h. The Facility Fire Protection Program and implementing procedures.
- i. An audit of the fire protection and loss prevention program implementation utilizing an outside independent fire protection consultant.
- j. The radiological environmental monitoring program and the results thereof.

The above audits may be conducted by the Quality Assurance Department or an independent consultant. Audit plans and final audit reports shall be reviewed by the OSR staff.

RECORDS AND REPORTS

6.5.2.4.4 Records of OSR activities shall be maintained. Reports of reviews and audits shall be prepared and distributed as indicated below:

- a. The results of reviews performed pursuant to Section 6.5.2.4.2 shall be reported to the Chief Nuclear Officer and President - Nuclear Business Unit at least monthly.
- b. Audit reports prepared pursuant to Section 6.5.2.4.3 shall be forwarded by the auditing organization to the Chief Nuclear Officer and President - Nuclear Business Unit and to the management positions responsible for the areas audited (1) within 30 days after completion of the audit for those audits conducted by the Quality Assurance Department, and (2) within 60 days after completion of the audit for those audits conducted by an independent consultant.

6.5.2.5 ON-SITE SAFETY REVIEW GROUP (SRG)

FUNCTION

6.5.2.5.1 The On-Site Safety Review Group (SRG) shall function to provide: the review of plant design and operating experience for potential opportunities to improve plant safety; the evaluation of plant operations and

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring.

6.8.2 Each procedure and administrative policy of 6.8.1 above, except 6.8.1.d and 6.8.1.e, and changes thereto, shall be reviewed and approved in accordance with Specification 6.5.1.6 or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d and 6.8.1.e shall be reviewed and approved in accordance with the Facility's Security and Emergency Plans or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 On-the-spot changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented and receives the same level of review and approval as the original procedure under Specification 6.5.3.2a within 14 days of implementation.



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PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. ¹⁶²
License No. DPR-75

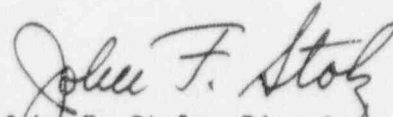
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated April 13, 1994, as supplemented December 6, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 162, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 11, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 162

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

Insert Pages

6-9

6-9

6-12

6-12

6-13

6-13

6-17

6-17

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

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- a. Review of: (1) Upper tier administrative procedures within the scope of Regulatory Guide 1.33 (2/78), and changes thereto and (2) Newly created procedures or changes to existing procedures that require a 10CFR50.59 safety evaluation as described in Section 6.5.3.2.d.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to Appendix "A" Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Review of the safety evaluations that have been completed under the provisions of 10CFR50.59.
 - f. Investigation of all violations of the Technical Specifications including the reports covering evaluation and recommendations to prevent recurrence.
 - g. Review of all REPORTABLE EVENTS.
 - h. Review of facility operations to detect potential nuclear safety hazards.
 - i. Performance of special reviews, investigations or analyses and reports thereon as requested by the General Manager - Salem Operations.
 - j. Not used.
 - k. Not used.
 - l. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10CFR50.59 safety evaluation.
 - m. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.4.2 The OSR staff shall review:

- a. The safety evaluations for changes to procedures, equipment, or systems; and tests or experiments completed under the provisions of 10CFR50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems; and tests or experiments that involve an unreviewed safety question as defined in 10CFR50.59;
- c. Proposed changes to Technical Specifications or to the Operating License;
- d. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- f. All REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Reports and meeting minutes of the Station Operations Review Committee.

AUDITS

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- b. The performance, training, and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the Criteria of Appendix B to 10CFR50.

ADMINISTRATIVE CONTROLS

AUDITS (continued)

- e. Not used.
- f. Not used.
- g. Any other area of facility operation considered appropriate by the Director - Quality Assurance and Nuclear Safety Review or the Chief Nuclear Officer and President - Nuclear Business Unit.
- h. The Facility Fire Protection Program and implementing procedures.
- i. An audit of the fire protection and loss prevention program implementation utilizing an outside independent fire protection consultant.
- j. The radiological environmental monitoring program and the results thereof.

The above audits may be conducted by the Quality Assurance Department or an independent consultant. Audit plans and final audit reports shall be reviewed by the OSR staff.

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6.5.2.5 ON-SITE SAFETY REVIEW GROUP (SRG)

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ADMINISTRATIVE CONTROLS

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