NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Nine Mile Point Unit #1	0 5 0 0 0 2 2 0	1 OF 0 2

TITLE (4)
Reactor Scram

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
												0 5 0 0 0
1 1	1 3	8 4	8 4	-	0 1 8	-	0 0	1 2	1 1	8 4		0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 4	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Robert Randall, Supervisor, Technical Support	3 1 5 3 4 9 - 2 4 4 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

During startup on November 13, 1984 the mechanical pressure regulator sent an erroneous signal to the bypass valve relay, (an instantaneous open and close signal) which caused the bypass valves to go open and cause a hi-level swell in the reactor, then to go closed and cause a shrinking level and subsequent scram of the reactor.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Nine Mile Point Unit #1	0 5 0 0 0 2 2 0	8 4	0 1 8	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TEXT

On November 13, 1984, during a startup with power at 4% thermal and 920 psig, the mechanical pressure regulator which controls the combined opening of the control and bypass valves failed. The mechanical pressure regulator has a set point range of 150 to 1060 psig, which makes this regulator suitable for use during reactor startup. At approximately 06:25 hrs. the mechanical pressure regulator did not respond to the designated set point. While the operators attempted to determine the cause of the failure the mechanical pressure regulator sent an erroneous signal which fully opened the bypass valves. At this time the pressure decreased and measured water level rose above 100". The mechanical pressure regulator then closed the bypass valves when the pressure decreased to 810 psig. The measured reactor water level then decreased to 40" as a result of the now increased pressure of 900 psig. The reactor scram was initiated as a result of low water level.

ASSESSMENT OF SAFETY CONSEQUENCES

There are no potential safety consequences resulting from this event since the plant engineered safety systems functioned as designed, and the measured water level swings were the result of shrink and swell, not actual inventory fluctuations.

CORRECTIVE ACTION

The mechanical pressure regulator was cleaned, lubricated and returned to service, and performed satisfactorily. The mechanical pressure regulator will be inspected during the next unit outage.

NMP-10133

NIAGARA MOHAWK POWER CORPORATION

NIAGARA MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

December 10, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

RE: Docket No. 50-220
LER 84-18

Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following Licensee Event Report:

LER 84-18 Which is being submitted in accordance with 10 CFR 50.73 (a) (2) (iv), "Any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS). However, actuation of an ESF, including the RPS, that resulted from and was part of the preplanned sequence during testing or reactor operation need not be reported."

This report was completed in the format designated in NUREG-1022, dated September 1983.

Very truly yours,

Thomas E. Lempges

Thomas E. Lempges
Vice President
Nuclear Generation

TEL/lo
Attachments
cc: Dr. Thomas E. Murley
Regional Administrator

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