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Southern Nuclear Operating Company
the southern electric system

J. D. Woodard
Vice President
Forley Project

May 1, 1992

Docket No. 50-364

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Joseph M. Farley Nuclear Plant - Unit 2
Steam Line Break Leakage Analysis of the Interim Plugging Criteria

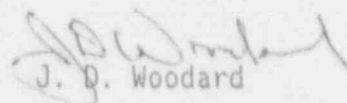
Gentlemen:

By letter dated April 1, 1992, the Nuclear Regulatory Commission issued Amendment 87 to the Unit 2 operating license regarding the steam generator tube interim plugging criteria. By letter dated March 27, 1992, Southern Nuclear Operating Company agreed to several additional requirements included in the amendment safety evaluation.

Attached is information concerning unexpected inspection findings and steam line break predicted leakage as required by the safety evaluation.

If there are any questions, please advise.

Respectfully submitted,


J. D. Woodard

Attachment

cc: Mr. S. D. Ebner
Mr. S. T. Hoffman
Mr. G. F. Maxwell

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ADD 1

Interim Plugging Criteria Analysis

1. NRC Request

The licensee shall inform the Staff, prior to Cycle 9 operation, of any unexpected inspection findings relative to the assumed characteristics of the flaws at the tube support plates. This includes any detectable circumferential indications or detectable indications outside the tube support plate thickness.

Southern Nuclear Response

The characteristics of the bobbin and RPC inspection results were typical of that found in earlier tube support plate inspections of the Farley steam generators. That is, no abnormal degradation, such as circumferentially oriented indications, was found in the 1992 Unit 2 inspection. Furthermore, no indications at tube support plates were identified which had cracks extending outside the tube support plates by either bobbin or RPC inspections.

2. NRC Request

The Staff's approval of the proposed interim repair limit is contingent on the licensee's being able to demonstrate that acceptance of all indications satisfying the 1.0 volt criterion will not create potential for leakage in excess of the 1.0 gpm licensing basis for a postulated steam line break (SLB) occurring at EOC 9. The licensee is requested to submit the results of the SLB leakage analysis prior to restart from the eighth refueling outage.

Southern Nuclear Response

Steam generator C, with highest voltage indications left in service, is the limiting steam generator for the SLB analysis. The projected SLB leakage is essentially zero, i.e., approximately 10^{-4} gpm.

As previously discussed with the Staff, indications above 1.0 volt which were left in service (no RPC flaws) were not included in SLB leakage analysis.