



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated September 20, 1995, as supplemented December 18 and 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 106 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance with full implementation within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy G. Colburn, Sr. Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: March 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

xxii  
3/4 0-1\*  
3/4 0-2  
3/4 0-3\*  
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3/4 6-7  
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3/4 0-5  
3/4 0-6  
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\*Overleaf page provided to maintain document completeness. No changes contained on these pages.

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### 3/4.0 APPLICABILITY

#### LIMITING CONDITION FOR OPERATION

3.0.1 Compliance with the Limiting Conditions for Operation contained in the succeeding Specifications is required during the OPERATIONAL CONDITIONS or other conditions specified therein; except that upon failure to meet the Limiting Conditions for Operation, the associated ACTION requirements shall be met.

3.0.2 Noncompliance with a Specification shall exist when the requirements of the Limiting Condition for Operation and associated ACTION requirements are not met within the specified time intervals. If the Limiting Condition for Operation is restored prior to expiration of the specified time intervals, completion of the ACTION requirements is not required.

3.0.3 When a Limiting Condition for Operation is not met, except as provided in the associated ACTION requirements, within one hour action shall be initiated to place the unit in an OPERATIONAL CONDITION in which the Specification does not apply by placing it, as applicable, in:

1. At least STARTUP within the next 6 hours,
2. At least HOT SHUTDOWN within the following 6 hours, and
3. At least COLD SHUTDOWN within the subsequent 24 hours.

Where corrective measures are completed that permit operation under the ACTION requirements, the ACTION may be taken in accordance with the specified time limits as measured from the time of failure to meet the Limiting Condition for Operation. Exceptions to these requirements are stated in the individual Specifications.

This Specification is not applicable in OPERATIONAL CONDITION 4 or 5.

3.0.4 Entry into an OPERATIONAL CONDITION or other specified condition shall not be made when the conditions for the Limiting Conditions for Operation are not met and the associated ACTION requires a shutdown if they are not met within a specified time interval. Entry into an OPERATIONAL CONDITION or other specified condition may be made in accordance with the ACTION requirements when conformance to them permits continued operation of the facility for an unlimited period of time. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements. Exceptions to these requirements are stated in the individual Specifications.



## APPLICABILITY

### SURVEILLANCE REQUIREMENTS

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4.0.1 Surveillance Requirements shall be met during the OPERATIONAL CONDITIONS or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.

4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval. For the purpose of the fifth refueling outage, those Surveillance Requirements listed on Table 4.0.2-1 and 4.0.2-2 are extended to the date specified in the table.

4.0.3 Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.2, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL CONDITIONS as required to comply with ACTION requirements.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, & 3 components shall be applicable as follows:

- a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable Addenda shall be applicable as follows in these Technical Specifications:

APPLICABILITY

SURVEILLANCE REQUIREMENTS (Continued)

<u>ASME Boiler and Pressure Vessel Code and applicable Addenda terminology for inservice inspection and testing activities</u>	<u>Required frequencies for performing inservice inspection and testing activities</u>
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days

- c. The provisions of Specification 4.0.2 are applicable to the above required frequencies for performing inservice inspection and testing activities.
- d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.
- e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
- f. The Inservice Inspection (NDE) Program for piping identified in NRC Generic Letter 88-01, dated January 25, 1988, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping", shall be performed in accordance with the staff positions on schedule, methods and personnel, and sample expansion included in this generic letter.

TABLE 4.0.2-1

SURVEILLANCE TEST INTERVALS EXTENDED TO OCTOBER 5, 1996

SURVEILLANCE REQUIREMENT

DESCRIPTION

4.1.3.1.4.a	Scram discharge vol. vent and drain valve operability
4.3.1.1, Table 4.3.1.1-1, Item 3	RPS Rx Steam Dome Press High cal.
4.3.1.1, Table 4.3.1.1-1, Item 4	RPS Rx Low Water Level - Level 3 cal
4.3.1.1, Table 4.3.1.1-1, Item 5	RPS MSIV Closure cal
4.3.1.1, Table 4.3.1.1-1, Item 6	RPS Main Steam Line Radiation High cal
4.3.1.1, Table 4.3.1.1-1, Item 7	RPS Drywell Pressure High cal
4.3.1.3 <sup>(a)</sup>	RPS Response Time Test
4.3.2.1, Table 4.3.2.1-1, Item 1.a.1	Pri Cont Isolation Actuation Rx Water Low Level - Level 3 cal
4.3.2.1, Table 4.3.2.1-1, Item 1.a.2	Pri Cont Isolation Actuation Rx Water Low Level - Level 2 cal
4.3.2.1, Table 4.3.2.1-1, Item 1.a.3	Pri Cont Isolation Actuation Rx Water Low Level - Level 1 cal
4.3.2.1, Table 4.3.2.1-1, Item 1.b	Pri Cont Isolation Actuation Drywell Press High cal
4.3.2.1, Table 4.3.2.1-1, Item 1.c.1	Pri Cont Isolation Actuation Main Steam Line Radiation High cal
4.3.2.1, Table 4.3.2.1-1, Item 1.c.2	Pri Cont Isolation Actuation Main Steam Line Press Low cal
4.3.2.1, Table 4.3.2.1-1, Item 1.d	Pri Cont Isolation Actuation Main Steam Line Tunnel Temp. High cal
4.3.2.1, Table 4.3.2.1-1, Item 1.e	Pri Cont Isolation Actuation Condenser Press High cal
4.3.2.1, Table 4.3.2.1-1, Item 1.f	Pri Cont Isolation Actuation Turbine Bldg. Area Temp. High cal
4.3.2.1, Table 4.3.2.1-1, Item 2.e	RWCU Isolation Rx Water Low Level - Level 2 channel cal
4.3.2.1, Table 4.3.2.1-1, Item 2.g	RWCU Manual Initiation channel functional test
4.3.2.1, Table 4.3.2.1-1, Item 3.a.1	RCIC Steam Line Flow High DP channel cal
4.3.2.1, Table 4.3.2.1-1, Item 3.a.2	RCIC Steam Line Flow High Time Delay cal
4.3.2.1, Table 4.3.2.1-1, Item 4.a.1	HPCI Steam Line Flow High DP cal
4.3.2.1, Table 4.3.2.1-1, Item 4.a.2	HPCI Steam Line Flow High Time Delay cal
4.3.2.1, Table 4.3.2.1-1, Item 4.e	HPCI Manual Initiation functional test
4.3.2.1, Table 4.3.2.1-1, Item 5.a	RHR S/D Cooling Rx Water Level Low - Level 3 cal
4.3.2.1, Table 4.3.2.1-1, Item 6.b	Sec. Cont. Isolation - Drywell Press High channel cal
4.3.2.3 <sup>(a)</sup>	Isolation Actuation Inst. System Response Time
4.3.3.1, Table 4.3.3.1-1, Item 1.b	CS Drywell Press High Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.b	LPCI Drywell Press High Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.f	LPCI Riser Differential Pressure High Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.g	LPCI Recirc. Pump Differential Pressure High Cal
4.3.3.1, Table 4.3.3.1-1, Item 3.b	HPCI Drywell Press High Cal
4.3.3.1, Table 4.3.3.1-1, Item 3.f	HPCI Manual Initiation
4.3.3.1, Table 4.3.3.1-1, Item 4.f	ADS RPV Low Level 3 Cal
4.3.3.1, Table 4.3.3.1-1, Item 4.i	ADS Manual Inhibit Functional Test
4.3.4, Table 4.3.4-1, Item 2	RPV Press High Cal (ATWS)
4.3.7.4.1, Table 4.3.7.4.-1, Item 1	RPV Press Cal - Remote Shutdown
4.3.7.5, Table 4.3.7.5-1, Item 1	RPV Press Cal Accident Mon.
4.3.7.5, Table 4.3.7.5-1, Item 11	SRV Position Indic Cal Accident Mon.
4.3.7.5, Table 4.3.7.5-1, Item 12	CTMT High Range Rad Monitoring Cal Accident Mon.
4.3.7.5, Table 4.3.7.5-1, Item 2.a	RPV Fuel Zone Level Cal Accident Mon
4.3.7.10.c	Loose Part Detection System Cal
4.3.9.1, Table 4.3.9.1-1, Item a	RPV High Water Level 8 Cal FW/Main Turbine Trip
4.3.9.2	FW/Main Turbine Trip LSFT
4.3.11.1, Table 4.3.11.1-1, Item 7	Alt S/D system Rx Water Level instrument operability
4.3.11.1, Table 4.3.11.1-1, Item 8	Alt S/D system Rx Press instrument operability
4.4.2.1.1	SRV Tail Pipe Pressure Switch Cal
4.4.2.1.2	SRV lift set point test
4.4.2.2.b	SRV Low Set Pressure setpoint Cal and LSFT
4.4.3.1.b	Drywell Sump Flow/Lvl Monitoring Cal
4.4.3.2.2.a	RCS Pressure Isol Valve Leak Test
4.5.1.d.2.a	ADS System Functional Test
4.6.1.2.b	Type B and C LLRT's
4.6.1.2.d	MSIV Leak Test
4.6.1.2.g	Hydrostatic Leak Test ECCS/RCIC Cont Isol Valves
4.6.1.4.d.3	MSIV LCS Press Inst. Cal and DP Calibration



TABLE 4.0.2-1

SURVEILLANCE TEST INTERVALS EXTENDED TO OCTOBER 5, 1996 Cont'd

<u>SURVEILLANCE REQUIREMENT</u>	<u>DESCRIPTION</u>
4.6.2.1.e	Suppression Chamber operability (visual inspection)
4.6.2.1.h	Suppression Chamber operability DW to torus bypass leak test
4.6.3.4	Instr. Excess Flow Check operability
4.6.3.5.b	TIP Explosive Squib operability test
4.6.4.1.b.2.a	Torus/Drywell vacuum breaker setpoint operability
4.6.4.1.b.2.b	Torus/Drywell vacuum breaker position indication cal
4.6.4.1.b.2.c	Torus/Drywell vacuum breaker switch opening gap
4.6.4.2.b.2.a	RB/Torus Vacuum Breaker operability (setpoint)
4.6.4.2.b.2.b	RB/Torus Vacuum Breaker operability (visual)
4.6.4.2.b.2.c	RB/Torus Vacuum Breaker position indication operability
4.7.11.4	Alternative Shutdown Control Circuit Functional Test
4.8.4.2.a.1.a	Primary Containment 4160 Volt Penetration Protective Relay Cal
4.8.4.2.a.1.b	Primary Containment 4160 Volt Penetration Protective Device Integrated Functional Test

TABLE NOTATIONS

- (a) The surveillance interval of channels within the same trip system required to be tested at least once every N times 18 months, where N is the total number of channels in the trip system, may be based upon the performance of the surveillance during the fifth refueling outage.

TABLE 4.0.2-2

SURVEILLANCE TEST INTERVALS EXTENDED TO END OF REFUELING OUTAGE 5

<u>SURVEILLANCE REQUIREMENT</u>	<u>DESCRIPTION</u>
4.1.3.5.b.2	CR Accumulator Integrity Test (Check Valve Leakage)
4.1.5.d.1	SLCS operability Manual Initiation
4.1.5.d.2	SLCS pump Relief Valve operability
4.1.5.d.3	SLCS flow path demonstration
4.3.1.1, Table 4.3.1.1-1, Item 11	RPS Rx Mode Switch shutdown position functional
4.3.1.2	RPS Logic System Function Test
4.3.2.1, Table 4.3.2.1-1, Item 1.h	Pri Cont Isolation Actuation Manual Initiation Functional
4.3.2.1, Table 4.3.2.1-1, Item 2.d	RVCU - SLCS initiation channel functional test
4.3.2.1, Table 4.3.2.1-1, Item 5.c	RHR S/D Cooling Rx manual initiation functional test
4.3.2.1, Table 4.3.2.1-1, Item 6.a	Sec. Cont. Isolation - Rx Water Low Level - Level 2 cal
4.3.2.2	Isolation Actuation Inst. LSFT
4.3.3.1, Table 4.3.3.1-1, Item 1.a	CS RPV Low Level 1 Cal
4.3.3.1, Table 4.3.3.1-1, Item 1.c	CS Rx Steam Dome Press Low Cal
4.3.3.1, Table 4.3.3.1-1, Item 1.d	CS Manual Initiation
4.3.3.1, Table 4.3.3.1-1, Item 2.a	LPCI RPV Low Level 1 Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.c	LPCI Rx Steam Dome Press Low Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.d	LPCI Rx Low Level 2 Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.e	LPCI Rx Steam Dome Press Low Cal
4.3.3.1, Table 4.3.3.1-1, Item 2.h	LPCI Manual Initiation
4.3.3.1, Table 4.3.3.1-1, Item 3.a	HPCI RPV Low Level 2 Cal
4.3.3.1, Table 4.3.3.1-1, Item 3.e	HPCI RPV High Level 8 Cal
4.3.3.1, Table 4.3.3.1-1, Item 4.a	ADS RPV Low Level 1 Cal
4.3.3.1, Table 4.3.3.1-1, Item 4.h	ADS Drywell Pressure High Bypass Timer
4.3.3.2	ECCS Logic System Functional Tests
4.3.3.3 <sup>(a)</sup>	ECCS Response Time Tests
4.3.4, Table 4.3.4-1, Item 1	RPV Low Water Level 2 Cal (ATWS)
4.3.4.2	ATWS Logic System Functional Test
4.3.5.1, Table 4.3.5.1-1, Item a	RPV Low Level 2 Cal (RCIC)
4.3.5.1, Table 4.3.5.1-1, Item b	RPV High Level 8 Cal (RCIC)
4.3.5.2	RCIC Logic System Functional Test
4.3.6, Table 4.3.6-1, Item 5.b	Scram Disc. Vol. Trip Bypass Funct. Test
4.3.6, Table 4.3.6-1, Item 7	Rx Mode Switch Shutdown Pos. Rod Block Funct. Test
4.3.7.4.1, Table 4.3.7.4-1, Item 2	RPV Level Cal - Remote Shutdown
4.3.7.5, Table 4.3.7.5-1, Item 16	CTMT Isolation Valve Position Cal Accident Mon
4.3.7.5, Table 4.3.7.5-1, Item 2.b	RPV Wide Range Level Cal Accident Mon
4.5.1.c.1	ECCS System Functional Test
4.6.3.2	Primary Containment Isol Valve operability
4.6.5.2.b	Secondary Containment Isolation Damper Actuation
4.7.1.2.b	ECCW Automatic Actuation
4.7.1.3.b	EESW Automatic Actuation
4.7.1.4.b	EDG Cooling Water Pump Automatic Actuation
4.7.2.1.c.1	CR Ventilation Filter Penetration
4.7.2.1.c.2	CR Ventilation Filter Charcoal Laboratory Analysis
4.7.2.1.c.3	CR Emergency Filtration System Flowrate
4.7.2.1.e.1	CR Ventilation Filter Pressure Drop
4.7.2.1.e.2	CR Emergency Filtration System Operational Mode Actuation
4.7.2.1.e.4	CR Emergency Makeup Inlet Heater Dissipation
4.7.5.e	Snubber Functional Test
4.8.1.1.2.e.1	EDG Inspection
4.8.1.1.2.e.2	EDG Load Rejection (1666 kW)
4.8.1.1.2.e.3	EDG Load Rejection (2850 kW)
4.8.1.1.2.e.4.a	EDG LOP Load Shedding
4.8.1.1.2.e.4.b	EDG LOP Auto Start and Load Sequencing

TABLE 4.0.2-2

SURVEILLANCE TEST INTERVALS EXTENDED TO END OF REFUELING OUTAGE 5 (Cont'd)

<u>SURVEILLANCE REQUIREMENT</u>	<u>DESCRIPTION</u>
4.8.1.1.2.e.5	EDG ECCS Auto Start
4.8.1.1.2.e.6.a	EDG LOP / ECCS Load Shedding
4.8.1.1.2.e.6.b	EDG LOP / ECCS Auto Start and Load Sequencing
4.8.1.1.2.e.7	EDG Non-essential Trip Bypass
4.8.1.1.2.e.8	EDG 24 Hour Run and Hot Fast Start.
4.8.1.1.2.e.9	EDG Auto Connect Load Verification
4.8.1.1.2.e.10	EDG Restoration of Offsite Power
4.8.1.1.2.e.11	EDG Auto Load Sequencer Timer
4.8.1.1.2.e.12.a	EDG 4160-volt ESF Bus Lockout
4.8.1.1.2.e.12.b	EDG Differential Trip Lockout
4.8.1.1.2.e.12.c	EDG Shutdown Relay Trip Lockout
4.8.2.1.c.3	130 VDC Battery Connections Resistance
4.8.2.1.c.4	130 VDC Battery Charger Functional Test
4.8.2.1.d	130 VDC Battery Capacity

TABLE NOTATIONS

- (a) The surveillance interval of channels within the same trip system required to be tested at least once every N times 18 months, where N is the total number of channels in the trip system, may be based upon the performance of the surveillance during the fifth refueling outage.

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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- g. ECCS and RCIC containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at least once per 18 months.
- h. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Specification 4.6.1.8.2.
- i. The provisions of Specification 4.0.2 are not applicable to Specifications 4.6.1.2a., 4.6.1.2b.\* and 4.6.1.2c.

\* Extension of Specification 4.6.1.2b per Table 4.0.2-1 for the fifth refueling outage is allowed.

CONTAINMENT SYSTEMS

PRIMARY CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

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3.6.1.3 Each primary containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to  $0.05 L_a$  at  $P_a$ , 56.5 psig.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2\*, and 3.

ACTION:

- a. With one primary containment air lock door inoperable:
  1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.
  2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
  3. Otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. With the primary containment air lock inoperable, except as a result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

\*See Special Test Exception 3.10.1.



## PLANT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- h. At least once per 36 months## by verifying that the sections of Control Room Emergency Filtration System duct listed in Table 4.7.2.1-1, when leak tested in accordance with ASME N510-1989# exhibit inleakage less than the acceptance criteria listed in Table 4.7.2.1-1 for the associated pressures.

4.7.2.2 The portions of the Control Room Emergency Filtration System duct listed below, which are accessible during normal operation, shall be visually inspected at least once per 366 days for cracking, debonding, or other abnormal degradation of the applied silicone sealant. Any such cracking, debonding, or other abnormal degradation shall be reported in accordance with Specification 6.9.2 within 14 days in a Special Report describing the findings and giving the intended course of action, including evaluation of and justification for continued plant operation.

- a. Normal intake between damper T4100F042 and the Control Room wall (Penetration V-430)
- b. Normal exhaust between damper T4100F044 and the Control Room wall (Penetration V-429)
- c. Discharge of recirculation fans T4100C047, 48 between the discharge flanges on filter train T4100D016 and the 5th Floor CCHVAC Equipment Room wall (Penetration V-504B)
- d. Division II supply plenum between the Control Room wall (Penetration V-431) and the 4th Floor Aux. Building ceiling (Penetration V-9014)
- e. Emergency intake between the discharge flange on filter train T4100D011 and the inlet flange on filter train T4100D016
- f. Recirculation duct between the 5th Floor CCHVAC Equipment Room wall (Penetration V-504A) and the inlet flange on filter train T4100D016

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# Tests performed in accordance with ANSI N510-1980 prior to the implementation of this requirement satisfy this requirement until the next required performance of the test.

##This surveillance requirement may be extended on a one-time basis to June 1, 1998.