

American Electric Power Corporation  
Donald C. Cook Nuclear Plant  
One Cook Place  
Bridgman, MI 49106  
(616) 465-5901



Donald C. Cook Plant Unit 2  
Docket No. 50-316  
License No. DPR-74

Subject: UNIT 2 REFUELING OUTAGE ISI

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington D.C., 20555

Attn.: Document Control Manager

February 14, 1996

The following is a synopsis of ISI tests and examinations scheduled to be performed during the 1996 Unit 2 Refueling Outage which is tentatively scheduled to start March 23, 1996.

Identification of specific valves (i.e. safety valves, check valves, etc.) can be obtained by referencing the Cook Plant Valve Test Program for the Second Ten Year Inspection Interval.

1. Ten-Year ultrasonic examination of the reactor vessel welds including 100% of the shell welds. (Will be performed by Southwest Research Institute)
2. Visual examination of the reactor internals including a special remote visual examination and mechanical agitation of selected former plate-core barrel bolts. (Will be performed by Master-Lee/B&W)
3. Eddy Current and ultrasonic examination of selected reactor head penetration welds in addition to repair of penetration #75. (Will be performed by Westinghouse)
4. Visual examination of disassembled check valves to check internal integrity and operability. (Will be performed by American Electric Power Personnel)
5. UT and surface examination of Class 1 and 2 pipe welds. (Will be performed by Southwest Research Institute and American Electric Power Personnel)
6. Visual Examination of Class 1, 2, and 3 component supports. (Will be performed by American Electric Power Personnel)

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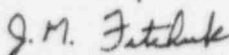
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Synopsis of Unit 1 ISI/IST  
Document Control Manager  
January 16, 1996

7. System pressure testing of Class 1, 2 and 3 systems. (Will be performed by American Electric Power Personnel)
8. Setpoint test three (3) pressurizer safety valves. (Will be performed by American Electric Power Personnel)
9. Setpoint test nineteen (14) class 2 and 3 safety valves. (Will be performed by American Electric Power Personnel)
10. Check valve internal examinations on a total of sixteen (17) valves in Class 1, 2 and 3 systems. (Will be performed American Electric Power Personnel)

Please see the attached listing for Class 1 and 2 welds scheduled to be examined per the Second Ten Year Interval Long Term Plan.

If there are any comments or questions or if further information is needed, please feel free to contact me at (616) 465-5901, extension 1410.

  
J.M. Fitchuk  
ISI/Performance Engineer

Attachments

c: see distribution list

c: H.J. Miller, Region III  
E.E. Fitzpatrick (w/o attachment)  
G. Charnoff (w/o attachment)  
J.R. Padgett - MI Public Service Commission (w/o attachment)  
D.R. Hahn - NFEM Section Chief (w/o attachment)  
W.T. Russell - NRR Director (w/o attachment)  
B. Bartlett - D.C. Cook Senior Resident Inspector (w/o attachment)  
J.R. Sampson - (w/o attachment)  
P.G. Schoepf - (w/o attachment)  
F.R. Pisarsky - (w/o attachment)  
P.A. Barrett - (w/o attachment)  
S.J. Brewer - (w/o attachment)  
File

D.C. Cook Unit 2  
1996 Refueling Outage

Inservice Examination of  
Class 1 and Class 2 Components

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 1

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE	ITEM NO METH	PROCEDURE	
-----						
<u>VESSEL CIRCUMFERENTIAL WELDS (FIG NO A-1)</u>						
000100	2-RPV-B UPPER SHELL TO MIDDLE SHELL	B-A	M-UT	B1.11	RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000200	2-RPV-C MIDDLE SHELL TO LOWER SHELL	B-A	M-UT	B1.11	RV-1/11-CSCL-5-DCC - U. CONT.	EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
000300	2-RPV-D LOWER SHELL TO LOWER HEAD	B-A	M-UT	B1.11	RV-3/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
<u>VESSEL LONGITUDINAL WELDS (FIG NO A-1)</u>						
000400	2-RPV-VA1 UPPER SHELL AT 22 DEG.	B-A	M-UT	B1.12	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000500	2-RPV-VA2 UPPER SHELL AT 113 DEG.	B-A	M-UT	B1.12	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000600	2-RPV-VA3 UPPER SHELL AT 247 DEG.	B-A	M-UT	B1.12	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000700	2-RPV-VB1 MIDDLE SHELL AT 170 DEG.	B-A	M-UT	B1.12	RV-1/11-CSCL-5-DCC - U. CONT.	EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
000800	2-RPV-VB2 MIDDLE SHELL AT 350 DEG.	B-A	M-UT	B1.12	RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 2

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY	EXAMINATION AREA	ASME SEC. XI	CATGY	NDE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/	
NUMBER	IDENTIFICATION	ITEM NO	METH	PROCEDURE	LOCATION	INSTRUCTIONS

VESSEL LONGITUDINAL WELDS (FIG NO A-1)

000900	2-RPV-VC1 LOWER SHELL AT 90 DEG.	B-A B1.12	M-UT		RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
001000	2-RPV-VC2 LOWER SHELL AT 270 DEG.	B-A B1.12	M-UT		RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.

REACTOR VESSEL LOWER HEAD MERIDIONAL WELDS (FIG NO A-1)

001740	2-LHM-03 MERIDIONAL WELD AT 129 DEG.	B-A B1.22	M-UT		RV-3/11-CSCL-5-DCC - U. CONT.	EXAMINE ACCESSIBLE LENGTH OF WELD IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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VESSEL TO FLANGE WELD (FIG NO A-1)

001800	2-RPV-A VESSEL TO FLANGE	B-A B1.30	M-UT		CSCL-5-DCC/CSCL-34-DCC - U. CONT.	EXAMINE 100% FROM SEAL SURFACE DURING 1ST PERIOD. EXAMINE 100% FROM VESSEL WALL DURING 3RD PERIOD. PERFORM EXAMINATIONS IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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CLOSURE HEAD TO FLANGE WELD (FIG NO A-2)

001900	2-CHC-01 HEAD TO FLANGE	B-A B1.40	MT UT		RV-2 B - U. CONT.	EXAMINE 33% (172") IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1 DURING EACH DESIGNATED OUTAGE.
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NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO A-1)

002000	2-N1-1 INLET NOZZLE TO SHELL AT 293 DEG.	B-D B3.90	M-UT		CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 3

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI CATGY NDE		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	ITEM NO	METH	PROCEDURE	
<u>NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO A-1)</u>					
002100	2-N1-0 SHELL TO OUTLET NOZZLE AT 338 DEG.	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE DURING 1ST PERIOD AND FROM THE VESSEL WALL DURING 3RD PERIOD. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002200	2-N2-I INLET NOZZLE TO SHELL AT 247 DEG.	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1. PERFORM SUPPLEMENTAL UT ON INDICATION DURING 2ND OUTAGE.
002300	2-N2-0 SHELL TO OUTLET NOZZLE AT 202 DEG.	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE DURING 1ST PERIOD AND FROM THE VESSEL WALL DURING 3RD PERIOD. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV.1.
002400	2-N3-I INLET NOZZLE TO SHELL AT 113 DEG.	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002500	2-N3-0 SHELL TO OUTLET NOZZLE AT 158 DEG.	B-D	M-UT	11-CSCL-5-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND FROM THE VESSEL WALL. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002600	2-N4-I INLET NOZZLE TO SHELL AT 67 DEG.	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.

ATTN: TAMMY

PAGE-01 FEB 13 1996  
AGENT-W3RAM ITIN

MUIERSTAUGH/MIKE  
FITCHUK/JIM

28 FEB 96 - WEDNESDAY

AMERICAN 4341 COACH CLASS EQUIP-TURBOPROP  
LV: SOUTH BEND 215P NONSTOP MILES- 84 CONFIRMED  
AR: CHICAGO/OHARE 205P ELAPSED TIME- :50  
OPERATED BY-SIMMONS AIRLINES  
SEAT- 9D

AMERICAN 1809 COACH CLASS EQUIP-MD-80 JET  
LV: CHICAGO/OHARE 250P NONSTOP MILES- 1041 CONFIRMED  
AR: SAN ANTONIO 536P ELAPSED TIME- 2:46

AVIS 1 INTERMED 2/4 DR DROP-01MAR CONFIRMED  
PICKUP-SAN ANTONIO SAN ANTONIO INTL APO  
RATE- 37.95 DAILY GUARANTEED  
MILEAGE-.00/MI CODE-2C  
CONFIRMATION-39934446US6

EMBASSY SUITES 02 NT/S - OUT 01MAR CONFIRMED  
ES SAN ANTONIO INTL 2 ROOM/S / 1 KING GUARANTEE-CREDIT CARD  
10110 HWY 281 NORTH RATE- 114.00 GUARANTEED  
SAN ANTONIO TX 78216 PHONE-210 525-9999  
FAX-210 525-0626

NAME-FITCHUK JIM  
CONFIRMATION-86094899

01 MAR 96 - FRIDAY

DELTA 552 COACH CLASS EQUIP-BOEING 727 JET  
LV: SAN ANTONIO 250P NONSTOP MILES- 1024 CONFIRMED  
AR: CINCINNATI 610P ELAPSED TIME- 2:20  
SEAT-36E

DELTA 1014 COACH CLASS EQUIP-BOEING 737 JET  
LV: CINCINNATI 710P NONSTOP MILES- 203 CONFIRMED  
AR: SOUTH BEND 807P ELAPSED TIME- :57  
SEAT-14B

PLEASE RECONFIRM FLIGHTS 72 HRS PRIOR TO FLIGHT DEPARTURE  
AIRLINE TICKETS ARE NON-REFUNDABLE AND NON-TRANSFERABLE  
\*\*\*\*THANK YOU FOR USING CRUISES AND TOURS\*\*\*\*

AIR TRANSPORTATION	398.00	TAX	9.00	TTL	407.00
		SUB TOTAL			407.00
		CREDIT CARD PAYMENT			407.00-





Donald C. Cook Plant Unit 2

Docket No. 50-315

License No. DPR-58

Subject: UNIT 2 REFUELING OUTAGE ISI

Arkwright Mutual Insurance Company

225 Wyman Street

P.O. Box 9198

Waltham, Mass. 02254-9198

January 22, 1996

Dear Mr. Muterspaugh:

The following is a synopsis of ISI tests and examinations scheduled to be performed during the 1996 Unit 2 Refueling Outage which is tentatively scheduled to start March 23, 1996.

Identification of specific valves (i.e. safety valves, check valves, etc.) can be obtained by referencing the Cook Plant Valve Test Program for the Second Ten Year Inspection Interval.

1. Ten-Year ultrasonic examination of the reactor vessel welds including 100% of the shell welds. (Will be performed by Southwest Research Institute)
2. Visual examination of the reactor internals including a special remote visual examination and mechanical agitation of selected former plate-core barrel bolts. (Will be performed by Master-Lee/B&W)
3. Eddy Current and ultrasonic examination of selected reactor head penetration welds in addition to repair of penetration #75. (Will be performed by Westinghouse)
4. Visual examination of disassembled check valves to check internal integrity and operability. (Will be performed by American Electric Power Personnel)
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6. Visual Examination of Class 1, 2, and 3 component supports. (Will be performed by American Electric Power Personnel)

Synopsis of Unit 1 ISI/IST  
Mr. M. Muterspaugh  
January 16, 1996

7. System pressure testing of Class 1, 2 and 3 systems. (Will be performed by American Electric Power Personnel)

8. Setpoint test three (3) pressurizer safety valves. (Will be performed by American Electric Power Personnel)

9. Setpoint test nineteen (19) class 2 and 3 safety valves. (Will be performed by American Electric Power Personnel)

10. Check valve internal examinations on a total of sixteen (16) valves in Class 1, 2 and 3 systems. (Will be performed American Electric Power Personnel)

Please see the attached the attached listing for Class 1 and 2 welds scheduled to be examined per the Second Ten Year Interval Long Term Plan.

If there are any comments or questions or if further information is needed, please feel free to contact me at (616) 465-5901, extension 1410.

J.M. Fitchuk  
ISI/Performance Engineer

Attachments

c: see distribution list

D.C. Cook Unit 2  
1996 Refueling Outage

Inservice Examination of  
Class 1 and Class 2 Components

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 1

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/		
NUMBER	IDENTIFICATION	CATGY NDE ITEM NO METH	PROCEDURE	LOCATION	INSTRUCTIONS

VESSEL CIRCUMFERENTIAL WELDS (FIG NO A-1)

000100	2-RPV-B UPPER SHELL TO MIDDLE SHELL	B-A B1.11	M-UT	RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000200	2-RPV-C MIDDLE SHELL TO LOWER SHELL	B-A B1.11	M-UT	RV-1/11-CSCL-5-DCC - U. CONT.	EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
000300	2-RPV-D LOWER SHELL TO LOWER HEAD	B-A B1.11	M-UT	RV-3/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.

VESSEL LONGITUDINAL WELDS (FIG NO A-1)

000400	2-RPV-VA1 UPPER SHELL AT 22 DEG.	B-A B1.12	M-UT	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000500	2-RPV-VA2 UPPER SHELL AT 113 DEG.	B-A B1.12	M-UT	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000600	2-RPV-VA3 UPPER SHELL AT 247 DEG.	B-A B1.12	M-UT	11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
000700	2-RPV-VB1 MIDDLE SHELL AT 170 DEG.	B-A B1.12	M-UT	RV-1/11-CSCL-5-DCC - U. CONT.	EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
000800	2-RPV-VB2 MIDDLE SHELL AT 350 DEG.	B-A B1.12	M-UT	RV-1/11-CSCL-5-DCC - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 2

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI	CATGY NDE		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	ITEM NO	METH	PROCEDURE			

VESSEL LONGITUDINAL WELDS (FIG NO A-1)

000900	2-RPV-VC1 LOWER SHELL AT 90 DEG.	B-A B1.12	M-UT		RV-1/11-CSCL-5-DCC - - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.
001000	2-RPV-VC2 LOWER SHELL AT 270 DEG.	B-A B1.12	M-UT		RV-1/11-CSCL-5-DCC - - U. CONT.	AUGMENTED EXAMINATION ON SHELL WELDS PER 10CFR50.55A.

REACTOR VESSEL LOWER HEAD MERIDIONAL WELDS (FIG NO A-1)

001740	2-LHM-03 MERIDIONAL WELD AT 129 DEG.	B-A B1.22	M-UT		RV-3/11-CSCL-5-DCC - - U. CONT.	EXAMINE ACCESSIBLE LENGTH OF WELD IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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VESSEL TO FLANGE WELD (FIG NO A-1)

001800	2-RPV-A VESSEL TO FLANGE	B-A B1.30	M-UT		CSCL-5-DCC/CSCL-34-DCC - - U. CONT.	EXAMINE 100% FROM SEAL SURFACE DURING 1ST PERIOD. EXAMINE 100% FROM VESSEL WALL DURING 3RD PERIOD. PERFORM EXAMINATIONS IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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CLOSURE HEAD TO FLANGE WELD (FIG NO A-2)

001900	2-CHC-01 HEAD TO FLANGE	B-A B1.40	MT UT		RV-2 B - - U. CONT.	EXAMINE 33% (172") IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1 DURING EACH DESIGNATED OUTAGE.
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NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO A-1)

002000	2-N1-I INLET NOZZLE TO SHELL AT 293 DEG.	B-D B3.90	M-UT		CSCL-5-DCC/CSCL-7-DCC - - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
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DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 3

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/	INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI	CATGY	NDE	LOCATION	
		ITEM NO	METH	PROCEDURE		
<u>NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO A-1)</u>						
002100	2-N1-0 SHELL TO OUTLET NOZZLE AT 338 DEG.	B-D	M-UT	B3.90	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE DURING 1ST PERIOD AND FROM THE VESSEL WALL DURING 3RD PERIOD. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002200	2-N2-I INLET NOZZLE TO SHELL AT 247 DEG.	B-D	M-UT	B3.90	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1. PERFORM SUPPLEMENTAL UT ON INDICATION DURING 2ND OUTAGE.
002300	2-N2-0 SHELL TO OUTLET NOZZLE AT 202 DEG.	B-D	M-UT	B3.90	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE DURING 1ST PERIOD AND FROM THE VESSEL WALL DURING 3RD PERIOD. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV.1.
002400	2-N3-I INLET NOZZLE TO SHELL AT 113 DEG.	B-D	M-UT	B3.90	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002500	2-N3-0 SHELL TO OUTLET NOZZLE AT 158 DEG.	B-D	M-UT	B3.90	11-CSCL-5-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND FROM THE VESSEL WALL. EXAMINE IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.
002600	2-N4-I INLET NOZZLE TO SHELL AT 67 DEG.	B-D	M-UT	B3.90	CSCL-5-DCC/CSCL-7-DCC - U. CONT.	EXAMINE FROM THE NOZZLE BORE AND THE VESSEL WALL IN ACCORDANCE WITH REG. GUIDE 1.150, REV. 1.

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 4

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/		INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY NDE ITEM NO METH	PROCEDURE	LOCATION	

NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS (FIG NO A-1)

002700	2-N4-O	B-D	M-UT	CSCL-5-DCC/CSCL-7-DCC	EXAMINE FROM THE NOZZLE BORE
	SHELL TO OUTLET NOZZLE AT 22	B3.90		-	AND FROM THE VESSEL WALL.
	DEG.			U. CONT.	EXAMINE IN ACCORDANCE WITH
					REG.GUIDE 1.150, REV.1.

NOZZLE INSIDE RADIUS SECTIONS (FIG NO A-3)

002800	2-N1-I-IRS	B-D	M-UT	IR-CSCL-B-DCC
	INLET NOZZLE AT 293 DEG.	B3.100		-
				U. CONT.
003000	2-N2-I-IRS	B-D	M-UT	IR-CSCL-B-DCC
	INLET NOZZLE AT 247 DEG.	B3.100		-
				U. CONT.
003200	2-N3-I-IRS	B-D	M-UT	IR-CSCL-B-DCC
	INLET NOZZLE AT 113 DEG.	B3.100		-
				U. CONT.
003300	2-N3-O-IRS	B-D	M-UT	IR-CSCL-B-DCC
	OUTLET NOZZLE AT 158 DEG.	B3.100		-
				U. CONT.
003400	2-N4-I-IRS	B-D	M-UT	IR-CSCL-B-DCC
	INLET NOZZLE AT 67 DEG.	B3.100		-
				U. CONT.
003500	2-N4-O-IRS	B-D	M-UT	IR-CSCL-B-DCC
	OUTLET NOZZLE AT 22 DEG.	B3.100		-
				U. CONT.

REACTOR VESSEL PENETRATIONS (FIG NO A-3)

003550	LOWER HEAD PENETRATIONS	B-E	VT-2		EXAMINE 25% (12 WELDS) DURING
	58 PENETRATIONS	B4.13		-	THE SYSTEM HYDROSTATIC TEST.
				CONT.	

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

PAGE: 5

REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA		ASME SEC. XI CATGY NDE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	ITEM NO METH	PROCEDURE		

REACTOR VESSEL PENETRATIONS (FIG NO A-2)

003600	CLOSURE HEAD CRD PENETRATIONS 78 PENETRATIONS	B-E B4.12	VT-2	- CONT.	EXAMINE 25% (20 WELDS) DURING THE SYSTEM HYDROSTATIC TEST.
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003610	CLOSURE HEAD VENT LINE 1 PENETRATION	B-E B4.11	VT-2	- CONT.	EXAMINE DURING THE SYSTEM HYDROSTATIC TEST.
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NOZZLE TO SAFE-END AND SAFE-END TO NOZZLE WELDS (FIG NO A-3)

003800	2-RPV-1-02 SAFE-END TO NOZZLE "COLD LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L1CL
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004000	2-RPV-2-02 SAFE-END TO NOZZLE "COLD LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L2CL
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004100	2-RPV-3-01 NOZZLE TO SAFE-END "HOT LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L3HL
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004200	2-RPV-3-02 SAFE-END TO NOZZLE "COLD LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L3CL
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004300	2-RPV-4-01 NOZZLE TO SAFE-END "HOT LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L4HL
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004400	2-RPV-4-02 SAFE-END TO NOZZLE "COLD LEG"	B-F B5.10	PT M-UT	PL-3.0-CSCL-4-DCC I - - SNDBX L4CL
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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR PRESSURE VESSEL AND CLOSURE HEAD

SUMMARY EXAMINATION AREA	ASME SEC. XI CATGY NDE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER IDENTIFICATION	ITEM NO METH PROCEDURE		

RPV INTERIOR SURFACES AND CORE SUPPORT STRUCTURES

006010	RPV UPPER INTERNALS & SURFACES B-N-1 VT-3 AREAS ACCESSIBLE DURING NORMAL B13.10 REFUELING	- U. CONT.	EXAMINE EACH PERIOD. AUGMENTED VT-1 ON BAFFLE PLATE FASTENERS IN ONE QUADRANT (90 DEG.) OF LOWER INTERNALS.
006021	RPV-CS-LUGS 1 THROUGH 6 B-N-2 VT-3 CORE SUPPORT LUGS B13.60	- U. CONT.	EXAMINE WHEN CORE BARREL IS REMOVED.
006040	2-RPV CORE SUPPORT STRUCTURE B-N-3 VT-3 (WHEN REMOVED) B13.70	- U. CONT.	EXAMINE WHEN REMOVED.

CONTROL ROD DRIVE HOUSING WELDS

006060	CONTROL ROD DRIVE HOUSING WELD B-0 PT 78 WELDS B14.10	B - U. CONT.	EXAMINE THE WELD ON 10% (8) OF THE PERIPHERAL CRDS.
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DATE: 02/13/96

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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PRESSURIZER (FIGURE A-4)

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE	ITEM NO METH	PROCEDURE	
<u>NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS</u>						
006970	14"-2-RC-21	B-D	MT		PL-3.0-CSCL-4-DCC	CODE RELIEF GRANTED FIRST INTERVAL FROM PERFORMING UT EXAMINATION FROM NOZZLE SIDE PROVIDED THAT (1) A FULL VEE VOLUMETRIC EXAMINATION IS PERFORMED FROM THE VESSEL SIDE WHEN ACCESSIBLE AND (2) SUPPLEMENTAL SURFACE.
	SURGE NOZZLE TO LOWER HEAD	B2.2	UT		B - -	
					CONT. PRT	

NOZZLE INSIDE RADIUS SECTIONS

007500	14"-2-RC-21-IRS	B-D	UT		IR-CS-33-DCC
	SURGE NOZZLE TO LOWER HEAD	B3.120			B - -
					CONT. PRT

HEATER CONNECTIONS/INSTRUMENT AND SAMPLE PENETRATIONS

007600	PRZ HEATER WELL PENETRATIONS	B-E	VT-2			EXAMINE ALL WELDS DURING SYSTEM HYDROSTATIC TEST.
	78 PENETRATIONS	B4.20			- -	
					CONT. PRT	
007700	PRZ INSTRUMENT AND SAMPLE NOZZ	B-E	VT-2			EXAMINE 25% (3 WELDS) DURING SYSTEM HYDROSTATIC TEST.
	9 PENETRATIONS	B4.13			- -	
					CONT. PRT	

NOZZLE TO SAFE-END AND SAFE-END TO NOZZLE WELDS

008500	2-PRZ-26	B-F	PT		3378032 (14-55-160-1.40)	AUGMENTED UT EXAMINATION REQUIRED USING 45 DEG. REFRACTED LONGITUDINAL WAVE PER NRC I.N. 90-30.
	SAFE-END TO NOZZLE	B5.40	UT		B - -	
					CONT. PRT	

LOWER HEAD TO SUPPORT SKIRT WELD

008505	2-PRZ-20	B-H	MT		PL-1.5-CS-78	WELD CONFIGURATION REQUIRES SURFACE EXAMINATION ON THE INSIDE AND OUTSIDE SURFACES. INSIDE SURFACE EXAMINATION TO BE PERFORMED DURING TEN-YEAR ISI TO COINCIDE WITH OTHER
	VESSEL SUPPORT	B8.20			B - -	
					CONT. PRT	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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STEAM GENERATOR NO. 21 (FIGURE A-5)

SUMMARY EXAMINATION AREA		ASME		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE	ITEM NO	METH	
		PROCEDURE				

CIRCUMFERENTIAL WELDS

008600	STM-21-01	B-B	UT	RV-3/DCC-41
	LOWER HEAD TO TUBE SHEET	B2.40		I - -
				CONT. L1

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
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STEAM GENERATOR NO. 22 (FIGURE A-5)

SUMMARY EXAMINATION AREA	ASME				
NUMBER IDENTIFICATION	SEC. XI	CATGY	NDE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
	ITEM NO	METH	PROCEDURE		
-----					

CIRCUMFERENTIAL WELDS

009500	STM-22-01	B-B	UT	RV-3/DCC-41
	LOWER HEAD TO TUBE SHEET	B2.40		I - -
				CONT. L2

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
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STEAM GENERATOR NO. 23 (FIGURE A-5)

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI	CATGY	NDE		
		ITEM NO	METH	PROCEDURE		
<hr/>						

CIRCUMFERENTIAL WELDS

010400	STM-23-01	B-B	UT	RV-3		
	LOWER HEAD TO TUBE SHEET	B2.40		I - -		
				CONT. L3		

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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STEAM GENERATOR NO. 24 (FIGURE A-5)

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE	PROCEDURE		
		ITEM NO	METH			
<u>NOZZLE INSIDE RADIUS SECTIONS</u>						
011400	STM-24-I-IRS	B-D	UT		IR-CSCL-24-DCC	
	INLET NOZZLE INSIDE	B3.140			B - -	
	RADIUSSECTION				CONT. L4	
011500	STM-24-O-IRS	B-D	UT		IR-CSCL-24-DCC	
	OUTLET NOZZLE INSIDE RADIUS	B3.140			B - -	
	SECTION				CONT. L4	
<u>NOZZLE TO ELBOW AND ELBOW TO NOZZLE WELDS</u>						
011600	STM-24-Q2	B-F	PT		CSS/CCS-MOCKUP-36-DCC	FULL VOLUMETRIC UT EXAMINATION
	ELBOW TO INLET NOZZLE	B5.70	UT		B - -	REQUIRED DUE TO NRC
					CONT. L4	COMMITMENT.
011700	STM-24-Q3	B-F	PT		CSS/CCS-MOCKUP-36-DCC	FULL VOLUMETRIC UT EXAMINATION
	OUTLET NOZZLE TO ELBOW	B5.70	UT		B - -	REQUIRED DUE TO NRC
					CONT. L4	COMMITMENT.

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME SEC. XI	CATGY NDE		PROCEDURE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	ITEM	NO	METH			
<u>LINE 2-RC-17 (FIG NO A-6)</u>							
014000	2-RC-17-01N 6-IN. NOZZLE (BRANCH CONNECTION)	B-J	PT			TO BE DETERMINED B - - CONT. L1	SCHEDULED PER CONDITION REPORT 95-0781.
015800	2-RC-17-08N 10-IN. NOZZLE (BRANCH CONNECTION)	B-J	PT			TO BE DETERMINED B - - CONT. L1	CODE RELIEF GRANTED FIRST INTERVAL FROM UT ON WELD DUE TO WELD GEOMETRY. SCHEDULED PER CONDITION REPORT 95-0781.
016200	2-RC-17-12 ELBOW TO SAFE-END	B-J	PT			37-CCSS-X-3.0-9-DCC I - - SNDBX L1CL	
<u>LINE 2-RC-18 (FIG NO A-7)</u>							
017000	2-RC-18-03N 14-IN. NOZZLE (BRANCH CONNECTION)	B-J	PT			TO BE DETERMINED B - - CONT. L2	SCHEDULED PER CONDITION REPORT 95-0781.
<u>LINE 2-RC-19 (FIG NO A-8)</u>							
019200	2-RC-19-01 SAFE-END TO PIPE	B-J	PT			37-CCSS-X-3.0-9-DCC I - - SNDBX L3HL	
<u>LINE 2-RC-22 (FIG NO A-11)</u>							
026500	2-RC-22-10 ELBOW TO PIPE	B-J	PT			3378027 (4-SS-120-.430) B - - PRESS. DH	
<u>LINE 2-RC-28 (FIG NO A-17)</u>							
034200	2-RC-28-22 PIPE TO TEE	B-J	UT			3378027 (4-SS-120-.430) B - - CONT. PRT	SCHEDULED IN RESPONSE TO NRCB 88-08 SUPP. 2. SEE WCAP-12143 SUPP. 1 FOR AUGMENTED UT REQUIREMENTS.

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME	CATGY NDE		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	ITEM NO	METH	PROCEDURE	LOCATION	
-----						
<u>LINE 2-RC-28 (FIG NO A-17A)</u>						
034300	2-RC-28-23 TEE TO PIPE	B-J B9.11	UT		3378027 (4-SS-120-.430) B - - CONT. PRT	SCHEDULED IN RESPONSE TO NRCB 88-08 SUPP. 2. SEE WCAP-12143 SUPP. 1 FOR AUGMENTED UT REQUIREMENTS.
034400	2-RC-28-24 PIPE TO ELBOW	B-J B9.11	UT		3378027 (4-SS-120-.430) B - - CONT. PRT	SCHEDULED IN RESPONSE TO NRCB 88-08 SUPP. 2. SEE WCAP-12143 SUPP. 1 FOR AUGMENTED UT REQUIREMENTS.
<u>LINE 2-RC-30 (FIG NO A-19)</u>						
041300	2-RC-30-06 PIPE TO TEE	B-J B9.11	UT		3378027 (4-SS-120-.430) B - - CONT. PRT	SCHEDULED IN RESPONSE TO NRCB 88-08 SUPP. 2. SEE WCAP-12143 SUPP. 1 FOR AUGMENTED UT REQUIREMENTS.
<u>LINE 2-RC-512 (FIG NO A-24)</u>						
058400	2-RC-512-01 BRANCH CONNECTION TO PIPE	B-J B9.40	PT		B - - CONT. L1	
058500	2-RC-512-02 PIPE TO ELBOW	B-J B9.40	PT		B - - CONT. L1	
058600	2-RC-512-03 ELBOW TO PIPE	B-J B9.40	PT		B - - CONT. L1	
058700	2-RC-512-04 PIPE TO ELBOW	B-J B9.40	PT		B - - CONT. L1	



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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI	CATGY	NDE		
		ITEM NO	METH	PROCEDURE		
<hr/>						
<u>LINE 2-RC-512 (FIG NO A-24)</u>						
058800	2-RC-512-05 ELBOW TO PIPE	B-J	PT		B - - CONT. L1	
058900	2-RC-512-06 PIPE TO ELBOW	B-J	PT		B - - CONT. L1	
059000	2-RC-512-07 ELBOW TO PIPE	B-J	PT		B - - CONT. L1	
059100	2-RC-512-08 PIPE TO ELBOW	B-J	PT		B - - CONT. L1	
059200	2-RC-512-09 ELBOW TO PIPE	B-J	PT		B - - CONT. L1	
059300	2-RC-512-10 PIPE TO VALVE	B-J	PT		B - - CONT. L1	
059400	2-RC-512-11 VALVE TO PIPE	B-J	PT		B - - CONT. L1	
059500	2-RC-512-12 PIPE TO ELBOW	B-J	PT		B - - CONT. L1	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE			
<u>LINE 2-RC-513 (FIG NO A-25)</u>						
064400	2-RC-513-17 PIPE TO TEE	B-J PT B9.40			B - - CONT. L1	
065000	2-RC-513-22 TEE TO PIPE	B-J PT B9.40			B - - CONT. L1	
065100	2-RC-513-23 PIPE TO MANIFOLD	B-J PT B9.40			B - - CONT. L1	
065200	2-RC-513-24 MANIFOLD TO PIPE	B-J PT B9.40			B - - CONT. L1	
065300	2-RC-513-25 PIPE TO ELBOW	B-J PT B9.40			B - - CONT. L1	
065400	2-RC-513-26 ELBOW TO PIPE	B-J PT B9.40			B - - CONT. L1	
065500	2-RC-513-27 PIPE TO REDUCER	B-J PT B9.21			B - - CONT. L1	
<u>LINE 2-RC-518 (FIG NO A-30)</u>						
080700	2-RC-518-15 VALVE TO PIPE	B-J PT B9.40			B - - CONT. L4	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE ITEM NO METH	PROCEDURE		
<u>LINE 2-RC-518 (FIG NO A-30)</u>						
080800	2-RC-518-16 PIPE TO ELBOW	B-J	PT B9.40		B - - CONT. L4	
080900	2-RC-518-17 ELBOW TO PIPE	B-J	PT B9.40		B - - CONT. L4	
081000	2-RC-518-18 PIPE TO TEE	B-J	PT B9.40		B - - CONT. L4	
081100	2-RC-518-19 TEE TO REDUCER	B-J	PT B9.40		B - - CONT. L4	
081200	2-RC-518-20 TEE TO PIPE	B-J	PT B9.40		B - - CONT. L4	
081300	2-RC-518-21 PIPE TO MANIFOLD	B-J	PT B9.40		B - - CONT. L4	
081700	2-RC-518-25 PIPE TO TEE	B-J	PT B9.40		B - - CONT. L4	
081800	2-RC-518-26 TEE TO REDUCER	B-J	PT B9.40		B - - CONT. L4	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/			INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY MDE ITEM NO METH	PROCEDURE	LOCATION		
-----						
<u>LINE 2-RC-518 (FIG NO A-30)</u>						
081900	2-RC-518-27 TEE TO PIPE	B-J B9.40	PT	B - - CONT. L4		
082000	2-RC-518-28 PIPE TO ELBOW	B-J B9.40	PT	B - - CONT. L4		
082100	2-RC-518-29 ELBOW TO PIPE	B-J B9.40	PT	B - - CONT. L4		
082200	2-RC-518-30 PIPE TO FLANGE	B-J B9.40	PT	B - - CONT. L4		
082400	2-RC-518-31 FLANGE TO PIPE	B-J B9.40	PT	B - - CONT. L4		
<u>LINE 2-RC-519 (FIG NO A-31)</u>						
084800	2-RC-519-17 PIPE TO TEE	B-J B9.40	PT	B - - CONT. L4		
084900	2-RC-519-18 TEE TO PIPE	B-J B9.40	PT	B - - CONT. L4		
085000	2-RC-519-19 PIPE TO VALVE	B-J B9.40	PT	B - - CONT. L4		

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECONW INTERVAL  
CLASS 1 COMPONENTS

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REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE		
<u>LINE 2-RC-519 (FIG NO A-31)</u>					
085100	2-RC-519-20 VALVE TO PIPE	B-J PT B9.40		B - - CONT. L4	
085200	2-RC-519-21 PIPE TO FLANGE	B-J PT B9.40		B - - CONT. L4	
085400	2-RC-519-22 TEE TO PIPE	B-J PT B9.40		B - - CONT. L4	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE		
<u>LINE 2-SI-58 (FIG NO A-35)</u>					
095300	2-SI-58-06 COLLAR TO PIPE	B-K-1 PT B10.10		B - - CONT. L3	100% UT NOT FEASIBLE DUE TO WELD CONFIGURATION.

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION			INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE			

LINE 2-CS-119 (FIG NO A-46)

129600	2-CS-119-12	B-J	PT			
	PIPE TO PIPE	B9.21		B	-	-
				CONT.	L4	

LINE 2-CS-121 (FIG NO A-48)

137200	2-CS-121-10	B-J	PT			
	PIPE TO BRANCH CONNECTION	B9.21		B	-	-
				CONT.	L4	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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VALVE PRESSURE RETAINING BOLTING (2" AND LESS)

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/		
NUMBER	IDENTIFICATION	SEC. XI CATGY	NDE	PROCEDURE	LOCATION	INSTRUCTIONS	
		ITEM NO	METH				
<u>LINE 2-SI-57 (FIG NO A-34)</u>							
220700	SI-170-L2	B-G-2	VT-1				
	FLANGE BOLTING	B7.70			B - -		
					CONT. L2		
<u>LINE 2-SI-59 (FIG NO A-36)</u>							
221200	SI-170-L4	B-G-2	VT-1				
	FLANGE BOLTING	B7.70			B - -		
					CONT. L4		
<u>LINE 2-SI-63 (FIG NO A-40)</u>							
221800	SI-158-L2	B-G-2	VT-1				
	FLANGE BOLTING	B7.70			B - -		
					CONT. L2		



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REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 1 COMPONENTS

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WALWORTH MOTOR-OPERATED GATE VALVES (8" NOMINAL)

SUMMARY	EXAMINATION AREA	ASME					
NUMBER	IDENTIFICATION	SEC. XI	CATGY	NDE	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS	
		ITEM NO	METH	PROCEDURE			

LINE 2-SI-60 (FIG NO A-37)

225300	SI-IMO-315	B-M-2	VT-3			ONE WALWORTH GATE VALVE TO BE
	VALVE BODY INTERNAL SURFACE	B12.50				EXAMINED BY I&M PERSONNEL
					ANNULUS	DURING THE INTERVAL. THE
						CHOICE DEPENDS ON MAINTENANCE
						REQUIREMENTS.

LINE 2-SI-62 (FIG NO A-39)

225400	SI-IMO-325	B-M-2	VT-3			
	VALVE BODY INTERNAL SURFACE	B12.50				
					ANNULUS	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
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CLASS 2 COMPONENTS

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STEAM GENERATOR NO. 24 (FIGURE B-4)

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE		
-----					

CIRCUMFERENTIAL WELDS

300390	STM-24-10	C-A	UT	PL-3.0-CS-22-DCC	
	SHELL TO HEAD	C1.20		B - -	
				CONT. L4	

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COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 2 COMPONENTS

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CHEMICAL AND VOLUME CONTROL TANK (FIGURE B-6)

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION			INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE			

INTEGRALLY WELDED VESSEL SUPPORTS

300670	2-CVCT-2VS-10, 11, & 12	C-C PT				
		C3.10				
				VCT RM		

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 2 COMPONENTS

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CTS HEAT EXCHANGER EAST (FIGURE B-7)

SUMMARY EXAMINATION AREA	ASME SEC. XI	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER IDENTIFICATION	CATGY NDE ITEM NO METH PROCEDURE		
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NOZZLE TO SHELL AND SHELL TO NOZZLE WELDS

300674	E-CTSHX-OUT	C-B	PT	
	SHELL TO NOZZLE	C2.11		
				E CTS HXRM

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
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BORON INJECTION TANK (FIGURE B-9)

SUMMARY EXAMINATION AREA		ASME			CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION	INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI	CATGY	NDE		
		ITEM NO	METH	PROCEDURE		

INTEGRALLY WELDED VESSEL SUPPORTS

300920	2-BIT-VS-3	C-C	MT			
		C3.10			-	-
					BIT RM	
300930	2-BIT-VS-4	C-C	MT			
		C3.10			-	-
					BIT RM	

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 2 COMPONENTS

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EMERGENCY CORE COOLING

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/		INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH PROCEDURE	LOCATION		

LINE 2-SI-62 (FIG NO B-19 & B-2)

311940	2-SI-62-02S	C-F PT	3378029 (8-SS-160-.81)		
	PIPE TO ELBOW	C5.21 UT	S - -		
			U. CONT.		

LINE 2-SI-77 (FIG NO B-22)

312450	2-SI-77-02S-PL 1 THROUGH 4	C-C PT			
	PIPE LUG	C3.20	S - -		
			ANNULUS		

LINE 2-RH-19 (FIG NO B-31)

314660	2-RH-19-26F	C-F PT			
	PIPE TO ELBOW	C5.11	L - -		
			VESTIBULE		

LINE 2-RH-21 (FIG NO B-33)

315330	2-RH-21-27S	C-F PT	3378029 (8-SS-160-.81)		
	ELBOW TO PIPE	C5.21 UT	L - -		
			VESTIBULE		

LINE 2-RH-31 (FIG NO B-37 & B-3)

315935	2-RH-31-10F-PS	C-C PT			
	PIPE SUPPORT	C3.20	I - -		
			ANNULUS		

### CONTAINMENT SPRAY SYSTEM

SUMMARY EXAMINATION AREA		ASME		CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	CATGY	NDE	PROCEDURE		
		ITEM NO	METH			
<u>LINE 2-CTS-10 (FIG NO B-44)</u>						
316540	2-CTS-10-12F VALVE TO PIPE	C-F C5.11	PT UT		10-SS-40-.365-26-DCC - - W CTS PPRM	7.5% AUGMENTED UT REQUIREMENT. TO BE EXAMINED EACH INTERVAL.
<u>LINE 2-CTS-11 (FIG NO B-45)</u>						
316970	2-CTS-11-22S ELBOW TO PIPE	C-F C5.11	PT		L - - VESTIBULE	
<u>LINE 2-CTS-13 (FIG NO B-47)</u>						
317360	2-CTS-13-09S TEE TO ELBOW	C-F C5.11	PT UT		10-SS-40-.365-26-DCC - - E CTS PPRM	7.5% AUGMENTED UT REQUIREMENT. TO BE EXAMINED EACH INTERVAL.
<u>LINE 2-CTS-14 (FIG NO B-48)</u>						
317660	2-CTS-14-01F NOZZLE TO REDUCER	C-F C5.11	PT		- - E CTS HXRM	
317680	2-CTS-14-03S PIPE TO ELBOW	C-F C5.11	PT UT		10-SS-40-.365-26-DCC - - E CTS HXRM	7.5% AUGMENTED UT REQUIREMENT. TO BE EXAMINED EACH INTERVAL.

DATE: 02/13/96  
REVISION: 0

COOK NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION SCHEDULE  
OUTAGE 2 (1996), THIRD PERIOD, SECOND INTERVAL  
CLASS 2 COMPONENTS

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FEEDWATER SYSTEM

SUMMARY EXAMINATION AREA		ASME	CALIBRATION BLOCK/ SIZE-SCHEDULE-ACCESS/ LOCATION		INSTRUCTIONS
NUMBER	IDENTIFICATION	SEC. XI CATGY NDE ITEM NO METH	PROCEDURE		

LINE 2-FW-76 (FIG NO B-56)

319590	2-FW-76-11S	C-F	MT		
	REDUCER TO ELBOW	C5.21	UT	B - -	
				CONT. L4	