



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 219
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated December 8, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 219, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented when the proper plant conditions can be established.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdorn, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 219

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-27a

3/4 3-27b

INSERT

3/4 3-27a

3/4 3-27b

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUES	
ii. RCS Loop ΔT Equivalent to Power > 50% RTP			R145
Coincident with Steam Generator Water Level--Low-Low (Adverse)	$\geq 15.0\%$ of narrow range instrument span	$\geq 14.4\%$ of narrow range instrument span	R155
and Containment Pressure (EAM)	≤ 0.5 psig	≤ 0.6 psig	R145
or Steam Generator Water Level--Low-Low (EAM)	$\geq 10.7\%$ of narrow range instrument span	$\geq 10.1\%$ of narrow range instrument span	R155
d. S.I.	See 1 above (all SI Setpoints)		
e. Loss of Power Start			
1. Voltage Sensors	≥ 5520 volts	≥ 5331 volts	
2. Load Shed Timer	1.25 seconds	1.25 \pm 0.25 seconds	
f. Trip of Main Feedwater Pumps	N.A.	N.A.	
g. Auxiliary Feedwater Suction Pressure-Low	≥ 3.21 psig (motor driven pump) ≥ 13.9 psig (turbine driven pump)	≥ 2.44 psig (motor driven pump) ≥ 12 psig (turbine driven pump)	R187
h. Auxiliary Feedwater Suction Transfer Time Delays	4 seconds (motor driven pump)	4 seconds \pm 0.4 seconds (motor driven pump)	
	5.5 seconds (turbine driven pump)	5.5 seconds \pm 0.55 seconds (turbine driven pump)	

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>	
7. LOSS OF POWER			R145
a. 6.9 kv Shutdown Board Undervoltage			
Loss of Voltage			
1. Voltage Sensors	≥ 5520 volts	≥ 5331 volts	
2. Diesel Generator Start and Load Shed Timer	1.25 seconds	1.25 ±0.25 seconds	
b. 6.9 kv Shutdown Board-Degraded Voltage			
1. Voltage Sensors	6456 volts	≥ 6403.5 volts (dropout) ≤ 6595.5 volts (reset)	
2. Diesel Generator Start and Load Shed Timer	≤ 300 seconds	≤ 370 seconds	
3. SI/Degraded Voltage Logic Enable Timer	9.5 seconds	9.5 ± 2.0 seconds	
8. ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INTERLOCKS			
a. Pressurizer Pressure			
1. Not P-11, Automatic Unblock of Safety Injection on Increasing Pressure	≤1970 psig	≤1975.2 psig	
2. P-11, Enable Manual Block of Safety Injection on Decreasing Pressure	≥1962 psig	≥1956.8 psig	R145



UNITED STATES
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TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated December 8, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 209, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented when proper plant conditions can be established.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 209

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-27a

3/4 3-27b

INSERT

3/4 3-27a

3/4 3-27b

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>	
ii. RCS Loop Δ T Equivalent to Power > 50% RTP			
Coincident with Steam Generator Water Level--Low-Low (Adverse) and Containment Pressure (EAM) or Steam Generator Water Level--Low-Low (EAM)	$\geq 15.0\%$ of narrow range instrument span	$\geq 14.4\%$ of narrow range instrument span	R132
	≤ 0.5 psig	≤ 0.6 psig	
	$\geq 10.7\%$ of narrow range instrument span	$\geq 10.1\%$ of narrow range instrument span	
d. S.I.	See 1 above (all SI Setpoints)		
e. Loss of Power Start			
1. Voltage Sensors	≥ 5520 volts	≥ 5331 volts	
2. Load Shed Timer	1.25 seconds	1.25 \pm 0.25 seconds	
f. Trip of Main Feedwater Pumps	N.A.	N.A.	
g. Auxiliary Feedwater Suction Pressure-Low	≥ 3.21 psig (motor driven pump) ≥ 13.9 psig (turbine driven pump)	≥ 2.44 psig (motor driven pump) ≥ 12 psig (turbine driven pump)	R175 R84
h. Auxiliary Feedwater Suction Transfer Time Delays	4 seconds (motor driven pump) 5.5 seconds (turbine driven pump)	4 seconds \pm 0.4 seconds (motor driven pump) 5.5 seconds \pm 0.55 seconds (turbine driven pump)	R116

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>	
7. LOSS OF POWER			R132
a. 6.9 kv Shutdown Board Undervoltage Loss of Voltage			
1. Voltage Sensors	≥ 5520 volts	≥ 5331 volts	
2. Diesel Generator Start and Load Shed Timer	1.25 seconds	1.25 ±0.25 seconds	
b. 6.9 kv Shutdown Board-Degraded Voltage			
1. Voltage Sensors	6456 volts	≥ 6403.5 volts (dropout) ≤ 6595.5 volts (reset)	
2. Diesel Generator Start and Load Shed Timer	≤300 seconds	≤ 370 seconds	
3. SI/Degraded Voltage Logic Enable Timer	9.5 seconds	9.5 ± 2.0 seconds	
8. ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INTERLOCKS			
a. Pressurizer Pressure			
1. Not P-11, Automatic Unblock of Safety Injection on Increasing Pressure	≤1970 psig	≤1975.2 psig	
2. P-11, Enable Manual Block of Safety Injection on Decreasing Pressure	≥1962 psig	≥1956.8 psig	R132