OPERATING DATA REPORT

DOCKET NO. 50-289
DATE November 15, 1984
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

OPERATING STATUS

1. UNIT NAME:	THREE MILE ISLAND UNIT 1	
2. REPORTING PERIOD:	OCTOBER ,1984.	
3. LICENSED THERMAL POWER (MWT):	2535.	
4. NAMEPLATE RATING (GROSS MWE):	871.	
5. DESIGN ELECTRICAL RATING(NET MWE):	819.	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE):	824.	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE):	776.	

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	745.	7320.	89113.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	35.0
20. UNIT AVAILABILITY FACTOR	0.0	0.0	35.0
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	34.2
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	32.7
23. UNIT FORCED OUTAGE RATE	100.0	100.0	61.9

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

IE24
1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE November 15, 198
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

MONTH: OCTOBER

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.	17	0.
2	0.	18	0.
3	0.	19	0.
4	0.	20	0.
5	0.	21	0.
6	0.	22	0.
7	0.	23	0.
8	0.	24	0.
9	0.	25	0.
10	0.	26	0.
11	0.	27	0.
12	0.	28	0.
13	0.	29	0.
14	0.	30	0.
15	0.	31	0.
16	0.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE November 15, 1984
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

REPORT MONTH October, 1984

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	84-10-84	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

OPERATING SUMMARY

The unit remained in cold shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The RCS was partially drained to permit OTSG work as described below.

MAJOR SAFETY RELATED MAINTENANCE

OTSG's RC-H-1A/B - Repairs to Once Through Steam Generator (OTSG) completed in October consisted of wire brushing, re-rolling and pull testing Westinghouse roll-type tube plugs in both generators. Two rolled plugs were removed by drilling out (1 in "B" upper and 1 in "A" upper). A total of thirty three new Westinghouse roll type tube plugs were installed in the generators to replace plugs that were removed during the previous pull testing evolution, and plugs that were removed by drilling out. The "A" and "B" OTSG's were drip tested and bubble tested with three tubes in the "A" OTSG identified as leaking and five tubes identified as leaking in the "B" OTSG. The leaking tubes were repaired by "hard" rolling the tubes approximately 10 inches below the top of the upper tubesheet. One B&W welded plug was also identified as leaking. The weld defect was ground out and the plug re-welded and satisfactorily "snoop" tested. The upper manway covers were installed (the lower manway covers were installed prior to bubble testing) after the channel heads were cleared of all tools, equipment and materials. The work tents were removed and the work platform on the "A" OTSG upper head was removed. The "B" OTSG work platform removal will be completed in November, 1984. Ventilation equipment was installed on upper handholes, placing both the "A" & "B" OTSG primary sides in dry lay-up. The secondary side of the OTSG's is in full wet layup. All major repair activities are complete.

Decay Heat Closed Cooling Pump DC-P-1B - the DC-P-1B mechanical seal installation work was completed in October, with the reinstallation of pump piping supports.

Emergency Diesel Generator EG-Y-1B - The annual Emergency Diesel inspection was completed on EG-Y-1B during the month of October. Various lube oil and fuel oil leaks were found during the inspection and were repaired. The fuel injector nozzles were removed and tested. Seven (7) injectors required replacement. The inspection of the exhaust manifold revealed a crack and the manifold was replaced. The inspection of the piston rings and cylinders were performed with satisfactory results. The #7 and #8 lower connecting rod bearings showed signs of wear and were replaced. The main bearing, camshaft torsional damper, governor drive, vertical drive and air blower inspections were performed, with all conditions satisfactory. The engine water system inspection showed four small leaks that were subsequently repaired. A hydro test was performed on the water system with satisfactory results. The lube oil and fuel oil filters were changed and the diesel ran for the 24 hour test with satisfactory results.

Emergency Diesel Generator EG-Y-1A - The annual Emergency Diesel Generator inspection on Eg-Y-1A commenced during October. The fuel injector nozzles, exhaust manifold, piston rings, camshaft, torsional dampers, governor drive, vertical drive, and air blower inspections were performed with satisfactory results. Five random main bearings were inspected, showing

minimal wear. The upper main thrust bearing inspection showed wear on the thrust surface and was replaced. Brass filings were found in the lube oil strainer during the inspection. An oil sample was taken for analysis. As a result of having found the brass filings, piston ring brushings, rod brushings, brushing inserts, crank shaft journals and lower thrust bearings were inspected with satisfactory results. The inspection will continue in the month of November.

Decay Heat Removal Pump DH-P-1A - Mechanical seal replacement of DH-P-1A is in progress with the pump internals removed from the casing and disassembly of pump internals. Work will continue in November.

Decay Heat Closed Cooling Pump DC-P-1A - Mechanical Seal Installation work on DC-P-1A started in October with the disassembly of the pump, balancing of the shaft and modifications to the pump internals. Work will continue in November. The pump is being modified to replace the existing packing with mechanical seals to reduce leakage.

Decay Heat Closed Cooling Cooler DC-C-1A - Work on DC-C-1A started with the removal of the end covers and tube cleaning in progress. Water box cleaning, end cover cleaning, and repairs to the water boxes will continue next month.

Limitorque Operator Dings Break Testing - Dings brakes on Limitorque valve operators were tested for degraded grid voltage conditions on valves DH-V-4A/B and DH-V-5A/B. All inspections/testing were satisfactory.

REFUELING INFORMATION REQUEST

1. Name of Facility:

   Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

   Unknown

3. Scheduled date for restart following refueling:

   Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

   (b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1988 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

GPU Nuclear

GPU Nuclear Corporation
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717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

November 15, 1984
5211-84-2283

Office of Management Information and Program Control
Attn: W. C. McDonald
c/o Distribution Services Branch DPC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. McDonald:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report - October 1984

Enclosed please find two (2) copies of the October 1984 Monthly Operating Report for Three Mile Island Nuclear Station Unit-1.

Sincerely,

R J Toole for

H. D. Hukill
Director, TMI-1

HDH:JGB:vjf

Attachments

cc: V. Stello
Dr. T. E. Murley

IE24
1/1