

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) CRYSTAL RIVER UNIT 3		DOCKET NUMBER (2) 0 5 0 0 0 3 0 2	PAGE (3) 1 OF 0 2
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TITLE (4)
Failure to Perform Required Radioactive Gaseous Effluent Sample

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
11	03	84	84	020	00	12	03	84	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

OPERATING MODE (9) 1

POWER LEVEL (10) 0 4 2

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.38(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and Text, NRC Form 388A)
<input type="checkbox"/> 20.405(a)(1)(iv)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME W. K. Bandhauer, Nuclear Safety Supervisor	TELEPHONE NUMBER 9 0 4 7 9 5 - 6 4 8 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	A	A	M	Q					
			B	1	5	0			Y

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a routine review of Surveillance Procedures, it was discovered that a sample analysis required by Technical Specifications was not performed within the required period. The sample analysis, an Auxiliary Building and Fuel Handling Area Exhaust Duct Monitor, RM-A2, Grab Sample analysis, is required to be performed between two and six hours following a change in power level exceeding 15% of rated thermal power within one hour. The power level change occurred at 2300 hours on November 2, 1984. A sample was analyzed on November 3, 1984 at 1741, 12 hours and 41 minutes later than allowed by Technical Specifications. A review of the recorder trace for RM-A2 following the power change showed no significant change in countrate. The cause of the event was personnel error. All licensed personnel will be re-instructed in the procedural requirements concerning notification of applicable departments.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 2 0	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

During a routine review of Surveillance Procedures it was discovered that a gas grab sample analysis required by Section 4.11.2.1.2 of Technical Specifications was not performed. The sample of concern is an Auxiliary Building and Fuel Handling Area Exhaust Duct Monitor, RM-A2 (IL), grab Sample analyzed for principal gamma emitters in accordance with Item C of Table 4.11-2 of Technical Specifications. This sample analysis is required to be performed between two and six hours following a change in power level exceeding 15% of rated thermal power within one hour.

On November 2, 1984 Crystal River Unit 3 was operating at approximately 92% reactor power generating 805 MWe. At 2300 hours a control rod stator (AA) failed causing the plant to runback power to approximately 60% in 3 minutes. Operations personnel informed the Chemistry Department of the power change but contrary to Special Conditions Surveillance Plan, SP-442, failed to inform them of the specific surveillances required. Due to this error only the gross equivalent iodine sample of the reactor coolant system was performed.

SAFETY CONSIDERATIONS

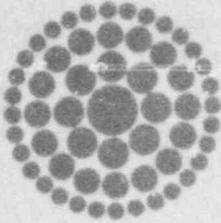
This event had no impact on the health and safety of the public. Inspection of the Environmental Radiation Monitor Recorder, RM-R6 (IL), showed that the count rate of RM-A2 did not change noticeably, indicating no significant change in the dose rate to the public from the effluent stream monitored by RM-A2. The results of the Reactor Coolant sample analysis taken as required for dose equivalent iodine showed no significant change in the source term, further indicating no significant change in the dose rate to the public.

CORRECTIVE ACTION

Licensed shift personnel will be reinstructed by December 31, 1984, in the requirements of the Special Conditions Surveillance Plan, SP-442.

PREVIOUS SIMILAR EVENTS

This is the second event concerning failure to perform an Auxiliary Building exhaust gas sample analysis within the specified time interval following a change in power level exceeding 15% of rated thermal power within an hour.



**Florida
Power**
CORPORATION

December 3, 1984
3F1284-01

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 84-020-00

Dear Sir:

Enclosed is Licensee Event Report (LER) No. 84-020-00 which is submitted in accordance with 10 CFR 50.73.

Should there be any questions, please contact this office.

Sincerely,

G. R. Westafer
Manager, Nuclear Operations
Licensing and Fuel Management

AEF:rw

Enclosure

cc: Mr. James P. O'Reilly
Regional Administrator, Region II
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, GA 30323

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