

NARRATIVE REPORT  
UNIT 1

October 1st

0000

Unit Refueling Outage in Progress.

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HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR October 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-494	05-07-84	Identified tagged by procedure all equipment installed on CRD Scram Discharge Volume Vents & Drains. Ref. DCR 83-032.
82-8041	10-15-84	Modified existing hangers/pipe supports on RHR & Plant Service Water System (MPL # 1P41). Ref. DCR 80-183.
84-5818	10-17-84	Installed Break Glass Stations around switches R26-M073 & R26-M074. Ref. DCR 83-032.

DOCKET NO. 50-321  
 DATE 11-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 10-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	745	7320	77447
12. Number of Hours Reactor was Critical	0	5638.8	55164
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	5474.9	51867.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	1215116.2	10936919.0
17. Gross Electrical Energy Generated (MWH)	0	3797550	35255720
18. Net Electrical Energy Generated (MWH)	-4033	3605148	33464511
19. Unit Service Factor	0	74.8	67.0
20. Unit Availability Factor	0	74.8	67.0
21. Unit Capacity Factor (Using MDC Net)	-0.7	65.5	57.4
22. Unit Capacity Factor (Using DER Net)	-0.7	63.4	55.6
23. Unit Forced Outage Rate	0	15.0	18.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling 10-1-84 10 week duration			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
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INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321  
 DATE: 11-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

MONTH 10-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net.)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net.)
1	-9	17	-4
2	-9	18	-4
3	-7	19	-4
4	-7	20	-4
5	-7	21	-4
6	-7	22	-5
7	-7	23	-4
8	-7	24	-4
9	-7	25	-4
10	-7	26	-4
11	-7	27	-4
12	-7	28	-4
13	-7	29	-4
14	-5	30	-4
15	-5	31	-4
16	-4		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October 1984

DOCKET NO. 50-321  
 UNIT NAME Hatch 1  
 DATE 11-06-84  
 COMPLETED BY Mike McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-72	84/10/01	S	745.00	C	2	N/A	RC	FUELXX	Unit Refueling Outage in progress.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuations  
 5-Load Reduction  
 9-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source



NARRATIVE REPORT  
UNIT 2

October 1st	0000	Still off-line following Rx. Scram due to loss of vacuum on 9-30-84.
	1400	Repairing valve 2N36-F00 5A extraction to 4th stage feedwater heater & also repairing RSLLV's 2N11-F00 6A&B.
	1933	Valve repairs complete. Increasing Rx. Power to 25% thermal via control rods.
October 2nd	0100	Movement of rods postponed because of problems with Rod Sequence Control System. Problems are being investigated.
	1145	Began pulling control rods. Ramping back to rated power.
	2345	Plant status at 582 MWe, 1771 MWT & still ramping to rated power.
October 3rd	2245	Plant status now at 94.5% CMWT, 2303 CMWT.
October 4th	1903	Automatic reactor scram on MSR hi level reading.
October 5th	0307	Reactor critical.
	1020	Generator tied to line following Rx. Scram on MSR hi level.
	1936	Holding power at 70% to run OD-1.
	2129	OD-1 completed.
	2300	Received high temp. alarm on 2B & 2D RHRSW Pumps. Maintenance was found to be working on these Unit 2 pumps when they should have been working on Unit 1 pumps.
October 6th	0445	Problem with RHRSW pumps resolved and unit is ramping to rated.
October 7th	0648	Plant status now at CMWT 2328 and 764 GMWe.

October 7th	15 30	Reactor scram on MSR hi level.
	2301	Reactor critical following scram on MSR hi level.
October 8th	0503	Generator tied to line following scram. Load increasing to approximately 15%. Load will be held at this level so that MSR testing may be performed for more problems.
October 9th	0600	Pulling control rods to increase load to approximately 200 GMWe.
	0903	Load now being increased to 500 GMWe.
	1320	Load being held at 500 GMWe due to APRM rod blocks and STA performing OD-1.
	1900	Starting to increase power again. Presently at 515 GMWe and increasing to approximately 540 GMWe by recirc flow on preconditioning ramp.
October 10th	2320	Plant status is now at 97% power with no major problems. Ramping to rated is still in progress.
October 11th	05 25	Daily turbine testing & RFPT testing is in progress.
October 12th	10 20	'B' recirc pump ran away to 100% speed pump is being backed down with manual crank.
	2340	Plant status back at 97% - 2363 CMWT and 783 MWe.
October 13th	2126	Load reduction in progress for rod adjustment & turbine testing. Load reduced by decreasing recirc flow.
October 14th	0212	Turbine testing and RFPT tests complete. Loading back to rated.
	0645	Load at 2433 CMWT and approximately 790 MWe. Load being increased to rated.
October 18th	1933	Shift noticed condenser vacuum decreasing. Problem is being investigated.
	2018	'Low Vacuum' alarm received and load is being decreased by recirc. flow to approximately 590 GMWe.

October 19th	0043	Power being increased to 700 MWe by recirc. flow.
	0518	Load increases stopped. Presently at 750 MWe.
	2110	Load reduction started by recirc. flow because of condenser vacuum problems.
October 20th	0815	Received orders for normal shutdown because main condenser vacuum broken.
	1318	Low Power level reactor scram.
October 21th	0500	Reactor critical following scram.
	1141	Generator tied to line. Ramping to rated by pulling of control rods. Condenser vacuum is being observed for more problems.
October 23th	0630	Plant status now at 97.2% CMWT 2368 and GMWe reading of 783.
October 27th	0020	Started reducing reactor load to 700 MWe for SRV valve testing (HNP-2-3901). Load presently at 792 GMWe.
	0235	Began increasing reactor load back to rated following completion of SRV's cycling. Load presently at 695 GMWe.
	0630	Load now at 99.1% CMWT of 2414. Load will remain close to rated for remaining time of the month.



HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR October 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
83-526	08-24-84	Pulled and terminated all cable & performed all internal wiring for Unit 2 Analog Transmitter Trip System. Also engraved all annunciator windows & nameplates. Ref. DCR 81-39.
83-527	08-25-84	Installed instrument racks, panels, instrumentation, air accumulates, piping, tubing, valves and supports required for installation of the Analog Transmitter Trip System. Ref. DCR 81-139.
83-528	08-24-84	Installed raceway & installed and fabricated raceway supports required to complete Analog Transmitter Trip System. Ref. DCR 81-139.
83-2421	08-23-84	Sealed Rosemount differential pressure transmitters Q2P70-FT-N020 A,B & N022A,B by procedure HNP-6954. Ref. DCR 81-112.
83-3552	07-31-84	Checked & replaced pressure relief valve 2T48-SV-3 on Nitrogen Storage Tank. Ref. DCR 84-179.
84-213	08-09-84	Replaced existing motor and HPCI Discharge Isol. Valve & Bypass valve 2E41-F007 & 2E41-F008 with new motor & actuator of class RH insulation Type SB to satisfy environmental qualification requirements of IEB Bulletin IEB-79-01B. Ref. DCR 81-177.
83-6143	07-11-84	Route and support conduit on Drywell Pneumatics system (MPL # 2P70). Ref. DCR 81-112.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR October 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-727	07-08-84	Modified pipe supports RCIC system (MPL # 2E51). Ref. DCR 82-257.
84-897	08-16-84	Modified pipe supports on Turbine Bypass Piping System (MPL # 2N37). Ref. DCR 82-257, Rev. 1.
84-899	06-08-84	Modified existing pipe supports 2E11-RSW-R17 & R25 on RHR Sys. Service Water Return. Ref. DCR 82-257, Rev. 1.
84-909	07-23-84	Modified existing pipe supports 2G41-FPC-A117 on Fuel Pool Cooling System. Ref. DCR 82-257, Rev. 1.
84-1049	07-23-84	Modified pipe supports on Turbine Bldg. - High Pressure Steam to steam jet air ejectors. (MPL # 2N11). Ref. DCR 82-257, Rev. 1.
84-1086	06-13-84	Terminated and redlined cables & wiring on the Drywell Pneumatic System Isolation Valve & Instruments Calibrated instruments 2P70-FT-N020A&B and 2P70-FT-N022A&B. Also installed supports and modified tubing and performed pneumatic test. Ref. DCR 81-112.
84-1176	05-16-84	Fabricated & installed instrument supports and tubing supports on Containment Atmosphere Recombiner Flow & Press. Transmitter. Removed existing flow and pressure transmitters & installed new Rosemount model flow and pressure transmitters. Also performed Pneumatic test on new tubing. Ref. DCR 83-37, Rev. 1.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR October 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-1477	05-30-84	Installed Conduit, Conduit Supports, Pull Boxes, Junction Boxes, Terminal Boxes, Removed existing conduit & rerouted existing cable for installation of instrumentation on Nitrogen Inerting System. Calibration of instrumentation was also performed. Ref. DCR 84-049, Rev. 1.
84-1478	07-13-84	Performed Installation and routing of air piping, fittings, valves, tubing, tube fittings, flex hose, air set, solenoid valve & pipe supports for 2P33-F016. Ref. DCR 81-165.
84-1485	08-18-84	Modified Pipe Supports 2N38-LFS-A34 on the High Pressure Steam Line to Reactor Feed Pump Turbine. Ref. DCR 82-257.
84-1526	07-23-84	Modified Existing pipe supports for MPL # 2C11-Control Rod Drive System. Ref. DCR 82-257, Rev. 1.
84-1812	08-17-84	Installed raceway supports, conduits, & cable channel for D.C. Switchgears Distribution Panels & Control Panels. Installed mounting pads for Switchgears 2R22-S018 & S019. Drilled Cove & set conduit sleeves. Also chipped floor for installation of ground cables. Installed Switchgears & D.C. Distribution panels 2R25-S129 & S130. Also installed Control Panels 2H21-P348 & P349. Terminated all rerouted cables.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR October 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-1921	08-06-84	Delete circuitry, conduit, thermostats & other associated equipment making up the old heat tracing system on H <sub>2</sub> O <sub>2</sub> Analyzers system - MPL # 2F33-P001A&B. Ref. DCR 81-165.
84-2257	07-18-84	Fabricated and installed start-up instrumentation brackets at locations A1, A3, B1 & B3. (MPL # 2B31). Ref. DCR 83-173.
84-2327	08-17-84	Installed additional fire barrier shielding around Remote Shutdown Panel 2C82-P001A&B. Ref. DCR 83-251.
84-2508	08-10-84	Installed & calibrated the resistance temperature detectors on start-up instrumentation (MPL # 2E11). Ref. DCR 83-173.



OPERATING DATA REPORT

DOCKET NO. 50-366  
 DATE 11-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: F. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 10-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	745	7320	45073
12. Number of Hours Reactor was Critical	718.5	1778.0	28849.2
13. Reactor Reserve Shutdown Hours	0.0	0	0
14. Hours Generator On-Line	680.0	1554.3	27342.2
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	142409.3	3139311	58355566
17. Gross Electrical Energy Generated (MWH)	472390	1027810	19199590
18. Net Electrical Energy Generated (MWH)	450751	955732	18244839
19. Unit Service Factor	91.7	21.3	60.7
20. Unit Availability Factor	91.7	21.3	60.7
21. Unit Capacity Factor (Using MDC Net)	81.0	17.5	54.1
22. Unit Capacity Factor (Using DER Net)	77.3	16.7	51.6
23. Unit Forced Outage Rate	8.5	8.6	13.0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
Recirc piping outage still in progress.			

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION



AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366  
 DATE: 11-10-84  
 COMPLETED BY: M. G. McBay  
 TELEPHONE (912) 367-7851

MONTH 10-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net.)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net.)
1	61	17	778
2	341	18	748
3	659	19	694
4	598	20	158
5	193	21	127
6	638	22	590
7	477	23	763
8	83	24	778
9	338	25	780
10	674	26	774
11	778	27	764
12	746	28	805
13	772	29	775
14	779	30	773
15	781	31	778
16	779		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 11-06-84  
 COMPLETED BY Mike McBay  
 TELEPHONE 912-367-7851

REPORT MONTH October 1984

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-17	84/10/01	F	10.68	A	3	N/A	HC	XXXXXX	Rx Scram on loss of condenser vacuum.
84-18	84/10/01	F	25.07	A	5	N/A	CH	VALVEX	Repair Ext. Valve to 4th stage feedwater heater & investigating problem with RSCS.
84-19	84/10/02	S	38.25	B	5	N/A	CH	VALVEX	Increasing power back to rated on preconditioning ramp.
84-20	84/10/04	F	15.28	A	3	N/A	HH	HTEXCH	Reactor scram on MSR hi level.
84-21	84/10/05	S	43.67	B	5	N/A	HH	HTEXCH	Increasing load back to rated.
84-22	84/10/07	F	13.55	A	3	N/A	HH	HTEXCH	Reactor scram on MSR hi level.
84-23	84/10/08	S	66.28	A	5	N/A	HH	HTEXCH	Holding load for MSR testing & preconditioning ramp to rated.

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A-Equipment Failure (Explain)  
B-Maintenance of Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup> Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuations  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October 1984

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 11-06-84  
 COMPLETED BY Mike McBay  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
84-24	84/10/12	F	13.33	A	5	N/A	CB	PUMPXX	Load reduction due to 'B' Recirc Pump running to 100% speed.
84-25	84/10/13	S	4.77	F	5	N/A	HA	TURBIN	Load reduction for Turbine test & rod pattern adjustment.
84-26	84/10/18	F	41.75	A	5	N/A	HC	XXXXXX	Load reduction due to condenser vacuum problems.
84-27	84/10/20	F	22.38	A	3	N/A	HC	XXXXXX	Reactor scram on low level due to the problems with condenser vacuum.
84-28	84/10/21	S	42.82	B	5	N/A	HC	XXXXXX	Scram recovery testing condenser vacuum being observed.
84-29	84/10/27	S	3.67	B	5	N/A	SH	VALVEX	SRV valve testing being performed.

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A-Equipment Failure (Explain)  
B-Maintenance of Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup> Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuations  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

Georgia Power Company  
Post Office Box 439  
Baxley, Georgia 31513  
Telephone 912 367-7781  
912 537-9444



Georgia Power

Edwin I. Hatch Nuclear Plant

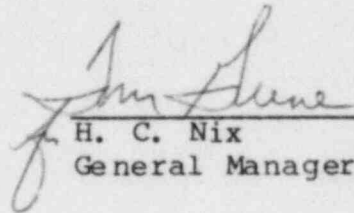
November 8, 1984  
GM-84-1025

PLANT E. I. HATCH  
NRC Monthly Operating Report

Office of Plans and Schedules  
Directorate of Licensing  
United States Nuclear Regulatory Commission  
Washington, D. C. 20545

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.

  
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H. C. Nix  
General Manager

*see [unclear]*  
HCN/CTJ/JAB/hh

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